



# Duquesne Light

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June 5, 1984

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Attn: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
Washington, DC 20555

Reference: Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
NUREG-0737, item II.B.3, Request for Additional Information

Gentlemen:

Your letter dated April 20, 1984 requested additional information on our Post-Accident Sampling System (PASS). Specifically, your request was for:

- our procedure to estimate the extent of core damage (II.B.3, criterion 2)
- the accuracy of the chloride analysis (II.B.3, criterion 2)
- information demonstrating applicability of procedures and instrumentation in the post-accident water chemistry and radiation environment (II.B.3, criterion 10)

On May 31, 1984, a conference call between Mr. P. Tam and Mr. P. Wu of the NRC and Mr. S. Sovick and Mr. V. Linnenbom was held in order to obtain clarification on the third of these three items. Mr. Wu explained that our response should address the accuracy of the PASS instruments when tested with the standard test matrix as to whether the instruments had met those accuracies to which we had committed. He was also questioning whether these instrument accuracies were demonstrated in the presence of a radiation field. As to this second concern, it was stated that the tests were not performed in the presence of this radiation field and that this was not a test criteria which had been made known at the time of equipment procurement or acceptance testing. However, the instruments were purchased with certification that they would function in a radiation field exceeding 10 E4 rads per gram of reactor coolant and were satisfactorily tested using the standard test matrix. Subsequent to this conference call we were advised that the reviewers in the NRC Chemical Engineering Branch had found our approach acceptable.

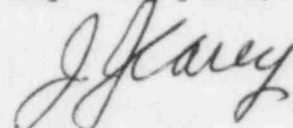
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The procedure to estimate core damage is currently being developed using the WOG core damage assessment methodology which was recently found acceptable by the NRC. In order to permit close-out of this NUREG-0737 item, Mr. Wu requested one submittal be made containing all the information requested by your letter. Agreement was reached during the conference call that all requested information would be provided upon completion of the core damage assessment procedure. Therefore, we will provide all the requested information by July 15, 1984.

If you have any questions concerning this submittal, please contact me or members of my staff.

Very truly yours,



J. J. Carey  
Vice President, Nuclear

cc: Mr. W. M. Troskoski, Resident Inspector  
U. S. Nuclear Regulatory Commission  
Beaver Valley Power Station  
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