

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station															DOCKET NUMBER (2) 0 5 0 0 0 3 9 5					PAGE (3) 1 OF 0 1					
TITLE (4) Main Steam Isolation																									
EVENT DATE (8)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	5	0	5	8	4	8	4	0	2	6	0	0	0	6	0	4	8	4	0	5	0	0	0		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																						
3			20.402(b)				20.408(a)				X 60.73(a)(2)(iv)				73.71(b)										
POWER LEVEL (10)			20.408(a)(1)(i)				60.38(a)(1)				60.73(a)(2)(v)				73.71(a)										
0 0 0			20.408(a)(1)(ii)				60.38(a)(2)				60.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)										
			20.408(a)(1)(iii)				60.73(a)(2)(i)				60.73(a)(2)(vii)(A)														
			20.408(a)(1)(iv)				60.73(a)(2)(ii)				60.73(a)(2)(viii)(B)														
			20.408(a)(1)(v)				60.73(a)(2)(iii)				60.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																									
NAME												TELEPHONE NUMBER													
A. R. Koon, Jr., Associate Manager, Reg. Compliance												8 0 3 3 4 5 - 5 2 0 9													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC															
X	J	E	I	S	V	W	1	2	0	N															
SUPPLEMENTAL REPORT EXPECTED (14)																									
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO		EXPECTED SUBMISSION DATE (15)													
												MONTH DAY YEAR													
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																									

On May 5, 1984, at 1929 hours, with the plant in Mode 3 (HOT STANDBY), a main steam isolation (Engineered Safety Features Actuation) occurred because of a hi steam flow signal coincident with a lo-lo Reactor Coolant System (RCS) Tave signal (553°F). The hi steam flow bistables had been tripped to allow maintenance on the C loop main steam flow transmitter IFT-494. Maintenance work was simultaneously being performed on the master main steam dump controller when the isolation occurred.

During installation of electronic cards into the controller, the steam dump manual/auto station generated a 50% demand signal to the steam dump system. The opening of the main steam dump valves cooled the RCS from 557°F to 551°F. The coincidence of the RCS lo-lo Tave and the tripped hi steam flow bistables satisfied the actuation logic for main steam isolation. The operators took prompt action to terminate the steam dump and re-established RCS normal operating temperature and pressure for Mode 3.

The Licensee has repeatedly attempted to induce a spurious master main steam dump controller output by alternating the installation and removal of electronic cards in the controller. The attempts failed to produce any spurious output; therefore, the Licensee considers this to be an isolated incident.

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PDR ADCK 05000395
S PDR

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

June 4, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555


SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 84-026

Dear Sir:

Please find attached Licensee Event Report #84-026 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

LEK:OWD/dwf
Attachment

cc: V. C. Summer
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