



Commonwealth Edison

Quad Cities Nuclear Power Station
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RLR-92-084

April 9, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Reference: Quad Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two

Enclosed is Licensee Event Report (LER) 92-006, Revision 01, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(i)(B). The licensee shall report any operation or condition prohibited by the plant's Technical Specification.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD CITIES NUCLEAR POWER STATION

R. L. Bax
Station Manager

RLB/TB/plm

Enclosure

cc: J. Schrage
T. Taylor
INPO Records Center
NRC Region III

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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Quad Cities Unit Two
 Title (4) .
 Docket Number (2) 0 | 5 | 0 | 0 | 0 | 2 | 6 | 5 |
 Page (3) 1 | 0 | 0 | 4

Missed Onsite Review of Temporary Procedure Due to Procedure Deficiency

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 1	7	9 2	9 2	0 0 6	0 1	0 4	0 7	9 2		0 5 0 0 0 1 1 0 5 0 0 0 1 1

OPERATING MODE (9) 1
 POWER LEVEL (10) 0 | 0 | %
 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in Abstract below and in Text)
20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name Terry Barber, Regulatory Assurance Ext. 3103
 TELEPHONE NUMBER AREA CODE 3 | 0 | 9 | 6 | 5 | 4 | - | 2 | 2 | 4 | 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) _____
 Yes (If yes, complete EXPECTED SUBMISSION DATE) X | NO

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

ABSTRACT:

On January 28, 1992, Quad Cities Unit Two was operating with the mode switch in the shutdown position, at 0 percent Reactor Core Thermal (RCT) power, with no fuel in the reactor vessel. Temporary Procedure (7370) 14 day on-site review time limit expired on this date.

On February 11, 1992, it was discovered that Temporary Procedure 7370 was not logged and filed in the Procedure Coordinator's Log. The original was lost or misplaced during routing for completion of the remaining three required approval signatures.

The cause of this event is a procedure deficiency.

A change to QAP 1100-5 "Temporary Procedures and Temporary Changes to Permanent Procedures", will be initiated to clarify the use of the closeout tracking form.

The Licensee Event Report (LER) is required due to the absence of the required on-site review within the 14 day period, required by Technical Specification 6.2.D.3. This report is being submitted in accordance with the requirements of the code of Federal Regulations Title 10, Part 50.73(a)(2)(i)(B).

FACILITY_NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						Page (3)
		Year		Sequential		Revision		
				Number		Number		
Quad Cities Unit Two	0 5 0 0 0 2 6 5	9 2	-	0 0 6	-	0 1	0 2 Of 0 4	
TEXT	Energy Industry Identification System (EIIS) codes are identified in the text as [XX]							

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power.

EVENT IDENTIFICATION: Missed Onsite Review of Temporary Procedure Due To Procedure Deficiency.

A. CONDITIONS PRIOR TO EVENT:

Unit: Two	Event Date: February 17, 1992	Event Time: 0000
Reactor Mode: 1	Mode Name: Shutdown	Power Level: 00%

This report was initiated by Deviation Report D-4-02-92-030.

Shutdown (1) - In this position a reactor is initiated, power to the control rod drives is removed, and the reactor protection trip systems have been deenergized for 10 seconds prior to permissive for manual reset.

B. DESCRIPTION OF EVENT:

At 10:11 hours on January 12, 1992, Quad Cities Unit Two fuel was completely unloaded from the reactor vessel. On January 28, 1992, Quad Cities Unit Two was operating with the mode switch in the shutdown position at 0 percent Rated Core Thermal (RCT) power, with no fuel in the reactor vessel.

Temporary Procedure 7370, "Withdrawal/Insertion of Control Rods With No Fuel in Reactor Vessel," was approved for use on January 13, 1992. Temporary Procedure 7370 was lost or misplaced during routing, and it is not known if the on-site review was completed within the 14 day period required by Technical Specification 6.2.D.3. This discrepancy was discovered on February 11, 1992. At that time the Procedure Coordinator notified the Technical Staff that Temporary Procedure 7370 was not logged and filed in the Procedure Coordinator's Log. A search for the Temporary Procedure was conducted but it was not found.

After being appropriately approved and stamped, this Temporary Procedure would normally have been placed in a basket in the Communication Center where it should have been picked up by an Assistant Technical Staff Supervisor. Per QAP 1100-7 "Approval and Authorization of Temporary Procedures and Temporary Changes, to Permanent Procedures", he verifies that the appropriate signatures have been received for Temporary Procedures that change intent. For Temporary Procedures that do not change intent, the Assistant Technical Staff Supervisor indicates the required signatures to complete the on-site review, and usually walks the Temporary Procedure around for those signatures. After obtaining the required signatures the procedure is forwarded to the Procedure Coordinator.

A copy of Temporary Procedure 7370, which was approved by two Senior Reactor Operators (SRO) on January 13, 1992, was on-site reviewed on February 11, 1992. This was 14 days past the 14 day period required by the Technical Specification.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev 2.0

7-15-171, NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			Page (3)		
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2-15-171, NAME (1)	0 5 0 0 0 2 6 5	9 2	0 0 6	0 1	0 2	OF	0 4

Regulatory Industry Identification System (EIS) codes are identified in the text as [XX]

There were no adverse effects caused by this event. The reactor core remained unloaded during this event. From January 12 through January 17, 1992, there was no movement of individual control rods. At 0510 hours on January 17, 1992, Out of Service (OOS) 4291 was initiated, which removed the Unit 2; 2A and 2B control rod drive pumps from service. This OOS has continued beyond February 11, 1992.

A copy of Temporary Procedure 7370, which was approved by two Senior Reactor Operators (SRO) on January 13, 1992, was on-site reviewed on February 11, 1992. This was 14 days past the 14 day period required by the Technical Specification.

There were no adverse effects caused by this event. The reactor core remained unloaded during this event. From January 12 through January 17, 1992, there was no movement of individual control rods. At 0510 hours on January 17, 1992, Out of Service (OOS) 4291 was initiated, which removed the Unit 2; 2A and 2B control rod drive pumps from service. This OOS has continued beyond February 11, 1992.

C. APPARENT CAUSE OF EVENT:

This Licensee Event Report (LER) is required due to the absence of the on-site review of a temporary procedure within a 14 day period, as required by Technical Specification 6.2.D.3. This report is submitted in accordance with the requirements of 10 CFR 50.73 (a)(2)(i)(B). The Licensee shall report any operations or condition prohibited by the plants' technical specifications.

The cause of this event is a procedure deficiency.

The original Temporary Procedure 7370 was lost or misplaced during the routing for completion of the remaining three required approval signatures. This was discovered after the 14 day on-site review time limit had expired.

QAP 1100-5 "Temporary Procedures and Temporary Changes to Permanent Procedures," Section 2C, requires the Procedure Coordinator to "ensure correct signatures and that the on-site review is completed within 14 days of approval." Section 2C also requires the Procedure Coordinator to "use Temporary Procedure Closeout Tracking Form, QAP 1100-T20, to track the Temporary Procedure through OSR Review Process." These changes were implemented to QAP 1100-5 by Revision 24, dated July 26, 1991.

Procedure Coordinator personnel changes in combination with the vague procedure made the deficiency apparent.

QAP 1100-5 Revision 24 was found to be vague in describing the use of the closeout tracking form, QAP 1100-T20.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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Quad Cities Unit Two	0 5 0 0 0 2 6 5	9 2	-	0 0 6	-	0 1	0 4 of 0 4

TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

D. SAFETY ANALYSIS OF EVENT:

The safety of the public and plant personnel was not affected during this event. During this event the reactor was operating at 0 percent RCT power, with no fuel in the reactor vessel. The reactor core remained unloaded during this event. Temporary Procedure 7370 was not utilized because there was no movement of individual control rods during this event.

E. CORRECTIVE ACTIONS:

The immediate corrective action was to search for the original Temporary Procedure. After the original Temporary Procedure could not be found, a copy of Temporary Procedure 7370 was onsite reviewed and signed on February 11, 1992.

A change to QAP 1100-5, "Temporary Procedures and Temporary Changes to Permanent Procedures," Section 2C, "Procedures Coordinator Responsibilities," will be initiated. The change will clarify the use of the close out tracking form, QAP 1100-T20 by the Procedure Coordinator (NTS #2652009203001).

F. PREVIOUS EVENTS:

A search was performed of previous events where Temporary Procedures were not on-site reviewed within the fourteen day time limit. One occurrence was identified.

Deviation Report
04-01-91-024

Temporary Procedure missed 14 day on-site
review time limit due to personnel error.

G. COMPONENT FAILURE DATA:

There was no component failure involved.