



Nuclear Group
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

April 13, 1992
ND3MNO:3283

Beaver Valley Power Station, Unit No. 2
Docket No. 50-412, License No. NPF-73
LER 92-003-00

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 92-003-00, 10 CFR 50.73.a.2.iv, "ESF Actuation - Feedwater Isolation due to Hi-Hi Level in the A Steam Generator".

Very truly yours,

T. P. Noonan
General Manager
Nuclear Operations

DSC/sl

Attachment

150003

9204160003 920413
PDR ADDCK 05000412
S PDR

TE22

April 13, 1992

ND3MNO:3283

Page two

cc: Mr. T. T. Martin, Regional Administrator
United States Nuclear Regulatory Commission
Region 1
475 Allendale Road
King of Prussia, PA 19406

C. A. Roteck, Ohio Edison
76 S. Main Street
Akron, OH 44308

Mr. A. DeAgazio, BVPS Licensing Project Manager
United States Nuclear Regulatory Commission
Washington, DC 20555

Larry Roessbach, Nuclear Regulatory Commission,
BVPS Senior Resident Inspector

Larry Beck
Centerior Energy
6200 Oak Tree Blvd.
Independence, Ohio 44101-4661

INFO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

G. E. Muckle,
Factory Mutual Engineering
680 Anderson Drive #BLD10
Pittsburgh, PA 15220-2773

Mr. Richard Janati
Department of Environmental Resources
P. O. Box 2063
16th Floor, Fulton Building
Harrisburg, PA 17120

Director, Safety Evaluation & Control
Virginia Electric & Power Co.
P.O. Box 26666
One James River Plaza
Richmond, VA 23261

W. Hartley
Virginia Power Company
5000 Dominion Blvd.
2SW Glenn Allen, VA 22060

J. M. Riddle
NUS Operating Service Corporation
Park West II
Cliff Mine Road
Pittsburgh, PA 15275

April 13, 1992

ND3MNO:3283

Page three

Bill Wegner, Consultant
23 Woodlawn Terrace
Fredricksburg, VA 22404

PAGE 73

0	5	0	0	0	4	1	2	1	OF	0	3
---	---	---	---	---	---	---	---	---	----	---	---

ESF Actuation - Feedwater Isolation due to Hi-Hi Level in the A Steam Generator

OPERATING MODE (8)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)				
4		20.402(b)	20.406(a)	X	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.406(a)(1)(i)	50.38(a)(1)		50.73(a)(2)(v)	73.71(c)
0100		20.406(a)(1)(ii)	50.38(a)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 3864)
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.3(a)(2)(viii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS IIR (12)

NAME	TELEPHONE NUMBER	
T.P. Noonan, General Manager Nuclear Operations	AREA CODE	
	4 1 2 6 4 3 - 1 2 5 8	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT 13

[illegible]

SUPPLEMENTAL REPORT EXPECTED 1/14/

ABSTRACT 11,000 to 14,000 species, i.e. approximately fifteen single words handwritten lines/16

MRC Form 368 (6-89)

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-620), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Beaver Valley Power Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 1 2 9 2 - 0 0 3 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Event

On 3/14/92, the station was in Operation Mode 4 (Hot Shutdown), and in the process of cooling down to begin refueling operations. Operators, in accordance with the shutdown procedures (Operating Manual procedure 20M-51.4, "Station Shutdown - Cooldown From Hot Standby to Cold Shutdown"), were maintaining the steam generator levels between 60 and 70 percent for an eight hour chemistry control soak period. Feed water to the steam generators was being provided from the condensate pumps via the bypass feedwater regulating valves (BFRV). The BFRVs had to be operated in manual, since their automatic control circuit would have attempted to maintain the steam generator level at the programmed level of 33 percent, which would be below the procedural level requirements of the chemistry soak.

At this time, operators were also placing the Residual Heat Removal (RHR) system in service. Operators were aligning the system and preparing to initiate RHR flow to support the plant cooldown.

At 1613 hours, the A steam generator level increased to 75 percent and initiated a feedwater isolation (FWI) signal. This FWI signal caused the three feedwater containment isolation valves FWS*HYV157A, B and C to close. All other components that receive a this FWI signal (the main turbine, the main feedwater pumps and the main feedwater regulating valves) had already been either removed from service or secured by the station shutdown procedures.

Cause of event

The shutdown procedure had the operators maintain the steam generator levels at 60 to 70 percent for the chemistry soak. This is above the high level deviation alarm setpoint (38 percent) and just below the feedwater isolation setpoint (75 percent). It has been determined that maintaining this level was not necessary and as such contributed to this event since the procedure was written to require this abnormal condition. Under these operating conditions, no alarm was available to alert the operators to the increasing level in the A steam generator. The operators were distracted while placing the RHR system into service and failed to maintain proper control of steam generators levels.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Beaver Valley Power Station Unit 2

0 5 0 0 0 4 1 2 9 2 - 0 0 3 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 305A's) (7)

Corrective Actions

- 1) At 1615 hours, level was reduced to less than 75 percent and then restored to the 60 to 70 percent range required by procedures. At 1622 hours, operators reopened the feedwater containment isolation valves that had been closed by the FWI signal.
- 2) The involved operator was counseled concerning this event.
- 3) The shutdown procedure is being revised to perform the steam generator soak at the normal level band so that the deviation alarm will be available to alert the operators to level increases.

Previous Similar Events

Review of station records revealed no recent similar events.

Reportability

The inadvertent feedwater isolation signal in this event actuated three containment isolation/engineered safety feature (ESF) valves and is being considered an unplanned ESF actuation. This event is therefore being reported in accordance with 10CFR50.73.a.2.iv.

Safety Implications

There were no safety implications due to this event. A feedwater isolation due to a Hi-Hi steam generator level is described in Beaver Valley Unit 2 UFSAR section 7.7.2.13, "Control systems not required for safety." This isolation does not serve any reactor safety function, but is provided to protect the main turbine blades from damage due to excessive moisture carryover that could result from high steam generator levels. The main turbine had already been removed from service prior to this event as part of the plant shutdown, therefore this equipment protective function was not required.