

The Light company

Houston Lighting & Power

South Texas Project Electric Generating Station P. O. Box 289 Wadsworth, Texas 77483

April 10, 1992
ST-HL-AE-4061
File No.: G02
10CFR50.71

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

South Texas Project
Units 1 & 2
Docket Nos. STN 50-498 & 50-499
Monthly Operating Reports for March, 1992

Pursuant to 10CFR50.71(a) and South Texas Project Electric
Generating Station (STPEGS) Technical Specification 6.9.1.5,
attached are the Monthly Operating Reports for March, 1992.

If you should have any questions on this matter, please
contact Mr. C. A. Ayala at (512) 972-8628.

William J. Jump
William J. Jump
Manager
Nuclear Licensing

MKJ/lf

Attachments: 1) STPEGS Unit 1 Monthly Operating Report -
March, 1992
2) STPEGS Unit 2 Monthly Operating Report -
March, 1992

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A Subsidiary of Houston Industries Incorporated

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Houston Lighting & Power Company
South Texas Project Electric Generating Station

ST-HL-AL- 4061
File No.: G02
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Revised 10/11/91

L4/NRC/

SOUTH TEXAS PROJECT
ELECTRIC GENERATING STATION
UNIT 1
MONTHLY OPERATING REPORT
MARCH 1992
HOUSTON LIGHTING AND POWER CO.
NRC DOCKET NO. 50-498
LICENSE NO. NPF-76

Reviewed By: <u>[Signature]</u>	<u>4-7-92</u>
Supervisor	Date
Reviewed By: <u>[Signature]</u>	<u>4-7-92</u>
Plant Engineering Manager	Date
Approved By: <u>[Signature]</u>	<u>4/8/92</u>
Plant Manager	Date

Monthly Summary

STPEGS Unit 1 began the reporting period at 100% reactor power.

On 3/14/92 at 1109 a reactor trip occurred due to a momentary false reactor coolant flow trip signal. Maintenance was being performed to calibrate a LOOP 2 Reactor Coolant Flow, Protection Set flow transmitter. Following calibration, when returning the transmitter to service, the high pressure side of the transmitter was opened first instead of the low pressure side to pressurize the transmitter. This caused a momentary low (below setpoint) differential pressure to be detected by two adjacent flow transmitters which initiated a reactor trip caused by a reactor coolant low flow indication.

The cause of the trip was attributed to failure to follow procedures. The unit was returned to service on 3/17/92 at 2359 and concluded the reporting period at full power.

OPERATING DATA REPORT

DOCKET NO. 50-498
UNIT 1
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

OPERATING STATUS

1. REPORTING PERIOD: 03/01-03/31 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800
MAX. DEPEND. CAPACITY (MWe-Net): 1250.6
DESIGN ELECTRICAL RATING (MWe-Net): 1250.6
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL.....	<u>668.6</u>	<u>2108.6</u>	<u>22128.5</u>
6. REACTOR RESERVE SHUTDOWN HOURS.....	<u>0</u>	<u>0</u>	<u>0</u>
7. HOURS GENERATOR ON LINE.....	<u>659.2</u>	<u>2099.2</u>	<u>21311.4</u>
8. UNIT RESERVE SHUTDOWN HOURS.....	<u>0</u>	<u>0</u>	<u>0</u>
9. GROSS THERMAL ENERGY GENERATED (MWt).....	<u>2470851</u>	<u>7960369</u>	<u>78114691</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)...	<u>834460</u>	<u>2718120</u>	<u>26426120</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH)....	<u>798605</u>	<u>2606156</u>	<u>24934525</u>
12. REACTOR SERVICE FACTOR.....	<u>89.9%</u>	<u>96.5%</u>	<u>70.1%</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>89.9%</u>	<u>96.5%</u>	<u>70.1%</u>
14. UNIT SERVICE FACTOR.....	<u>88.6%</u>	<u>96.1%</u>	<u>67.5%</u>
15. UNIT AVAILABILITY FACTOR.....	<u>88.6%</u>	<u>96.1%</u>	<u>67.5%</u>
16. UNIT CAPACITY FACTOR (Using MDC).....	<u>85.8%</u>	<u>95.4%</u>	<u>63.2%</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)...	<u>85.8%</u>	<u>95.4%</u>	<u>63.2%</u>
18. UNIT FORCED OUTAGE RATE.....	<u>11.4%</u>	<u>3.9%</u>	<u>13.4%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	<p>Scheduled 8 day maintenance outage to begin April 6, 1992. Refueling and scheduled maintenance outage to begin September 19, 1992.</p>		
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	<u>N/A</u>		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-498
UNIT 1
DATE Apr. 2, 1992
COMPLETED BY A. P. Kent
TELEPHONE 512/972-7786

MONTH MARCH

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>1258</u>
2	<u>1257</u>
3	<u>1258</u>
4	<u>1258</u>
5	<u>1258</u>
6	<u>1255</u>
7	<u>1253</u>
8	<u>1252</u>
9	<u>1254</u>
10	<u>1256</u>
11	<u>1255</u>
12	<u>1258</u>
13	<u>1256</u>
14	<u>555</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0</u>
18	<u>652</u>
19	<u>1215</u>
20	<u>1217</u>
21	<u>1217</u>
22	<u>1219</u>
23	<u>1221</u>
24	<u>1221</u>
25	<u>1221</u>
26	<u>1219</u>
27	<u>1207</u>
28	<u>1131</u>
29	<u>1216</u>
30	<u>1219</u>
31	<u>1218</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-4

UNIT 1

DATE APR 1

COMPLETED BY A.P.

TELEPHONE 512/9

REPORT MONTH MARCH

No.	Date	Type	Duration (Hours)	Reason	2	Method of Shutting Down Reactor	3	Licensee Event Report #	4	System Code	5	Component Code	Cause & Corrective Action to Prevent Recurrence
92-01	920314	F	84.8	G		3		1-92-003	AB			FI1	<p>A reactor trip occurred due to a momentary false reactor coolant flow trip signal. Maintenance was being performed to calibrate a LOOP 2 Reactor Coolant Flow Protection Set flow transmitter. Following calibration, when returning the transmitter to service, the high side of the transmitter was opened first instead of the low pressure side to pressurize the transmitter. This caused a momentary low (below setpoint) differential pressure to be detected by two adjacent flow transmitters which initiated a reactor trip caused by a reactor coolant low flow indication.</p> <p>Corrective actions to prevent recurrence will be addressed in the LER.</p>

1 F: Forced
S: Scheduled

2 Reason:

- A-Equipment Failure (Explain)
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Exam
- F-Administrative
- G-Operational Error (Explain)

3 Method:

- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Cont. of Existing
 Unlge
- 5-Reduction
- 9-Other

4 IEEE 805-1983

5 IEEE 803A-1983

ATTACHMENT 1
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PORVs and Safety Valves Summary

There were no PORV or Safety Valves challenged during the reporting period.

SOUTH TEXAS PROJECT
ELECTRIC GENERATING STATION
UNIT 2
MONTHLY OPERATING REPORT
MARCH 1992
HOUSTON LIGHTING AND POWER CO.
NRC DOCKET NO. 50-499
LICENSE NO. NPF-80

Reviewed By: *[Signature]*
Supervisor

4-7-92
Date

Reviewed By: *[Signature]*
Plant Engineering Manager

4-7-92
Date

Approved By: *[Signature]*
Plant Manager

4/8/92
Date

Monthly Summary

STPEGS Unit 2 began the reporting period at 100% reactor power.

On 3/9/92 Steam Generator Feedwater Pump Turbine 23 was tripped due to an electro-hydraulic control fluid leak on the line to the low pressure stop valve. The fluid leak caused the stop valve to stick in the open position. A weld had cracked causing a leak to develop. The weld and a portion of the line was replaced. Two hydraulic actuators were replaced to allow the stop valve to return to service. During the loss of the feedwater pump turbine, the Master Controller was responding erratically. Reactor power was reduced to 49% to minimize the possibility of a unit trip during troubleshooting. A spike was detected on an auxiliary relay card in the 7300 cabinet. The card was replaced and the unit was returned to full power on 3/12/92.

On 3/30/92 Steam Generator Feedwater Pump Turbine 22 tripped on low oil pressure when a main oil pump tripped due to a fault in the cabling. Reactor power was reduced to 48% while investigating the cause of the pump trip. The cable at the oil pump was repaired. The unit completed the reporting period with the unit approaching full power.

OPERATING DATA REPORT

DOCKET NO. 50-499
UNIT 2
DATE Apr 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

OPERATING STATUS

1. REPORTING PERIOD: 03/01-03/31 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800
MAX. DEPEND. CAPACITY (MWe-Net): 1250.6
DESIGN ELECTRICAL RATING (MWe-Net): 1250.6
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL.....	744.0	2035.6	17458.2
6. REACTOR RESERVE SHUTDOWN HOURS.....	0	0	0
7. HOURS GENERATOR ON LINE.....	744.0	1998.3	16480.5
8. UNIT RESERVE SHUTDOWN HOURS.....	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWt)....	2663411	7366291	59759562
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)...	408680	2513490	20180850
11. NET ELECTRICAL ENERGY GENERATED (MWH)....	870309	2407855	19111858
12. REACTOR SERVICE FACTOR.....	100.0%	93.2%	71.5%
13. REACTOR AVAILABILITY FACTOR.....	100.0%	93.2%	71.5%
14. UNIT SERVICE FACTOR.....	100.0%	91.5%	67.5%
15. UNIT AVAILABILITY FACTOR.....	100.0%	91.5%	67.5%
16. UNIT CAPACITY FACTOR (Using MDC).....	93.5%	88.2%	62.6%
17. UNIT CAPACITY FACTOR (Using Design MWe)...	93.5%	88.2%	62.6%
18. UNIT FORCED OUTAGE RATE.....	0.0%	8.5%	15.5%
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): N/A			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A			

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-499

UNIT 2

DATE Apr. 2, 1992

COMPLETED BY A. P. Kent

TELEPHONE 512/972-7786

MONTH MARCH

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	1251
2	1254
3	1255
4	1256
5	1255
6	1254
7	1255
8	1255
9	976
10	543
11	545
12	962
13	1252
14	1251
15	1249
16	1249

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	1252
18	1255
19	1257
20	1255
21	1256
22	1256
23	1252
24	1250
25	1250
26	1255
27	1256
28	1253
29	1256
30	1252
31	649

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-499UNIT 2DATE Apr. 2, 1992COMPLETED BY A.P. KentTELEPHONE 512/972-7786REPORT MONTH MARCH

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
92-03	920309	F	0.0	B	S	N/A	JC	P	Steam Generator Feedwater Pump Turbine 23 was tripped due to an electro-hydraulic control fluid leak on the line to the low pressure stop valve. The fluid leak caused the stop valve to stick in the open position. A weld had cracked causing a leak to develop. The weld and a portion of the line was replaced. Two hydraulic actuators were replaced to allow the stop valve to return to service. During the loss of the feedwater pump turbine, the Master Controller was responding erratically. The reactor was reduced to 49% to minimize the possibility of a unit trip during troubleshooting. A spike was detected on an auxiliary relay card in the 7300 cabinet. The card was replaced.

¹
F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Cont. of Existing
Outage
5-Reduction
9-Other

⁴ IEEE 805-1983

⁵ IEEE 803A-1983

ATTACHMENT 2
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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-499
 UNIT 2
 DATE Apr. 2, 1992
 COMPLETED BY A.P. Kent
 TELEPHONE 512/972-7786

REPORT MONTH MARCH

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
92-04	920330	F	0.0	B	5	N/A	SJ	P	Steam Generator Feedwater Pump Turbine 22 tripped on low oil pressure when a main oil pump tripped due to a fault in the cabling. Reactor power was reduced to 48% while investigating the cause of the pump trip. The cable at the oil pump was repaired.

¹
 F: Forced
 S: Scheduled

² Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Exam
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³ Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Cont. of Existing
 Outage
 5-Reduction
 9-Other

⁴ IEEE 805-1983

⁵ IEEE 803A-1983

ATTACHMENT 2
 ST-HI-AE-4061
 PAGE 6 OF 7

PORVs and Safety Valves Summary

There were no PORV or Safety Valves challenged during the reporting period.