

TU ELECTRIC

Log # TXX-91270  
File # 10010  
915.6  
Ref. # 10CFR50.46

William J. Cahill, Jr.  
Executive Vice President

July 31, 1991

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES) - UNIT 2  
DOCKET NO. 50-446  
PEAK CLADDING TEMPERATURE CHANGES GREATER THAN 50°F

REF: 1) Letter logged TXX-6444, dated May 18, 1987, from,  
W. G. Council to the NRC

Gentlemen:

Westinghouse Electric Corporation notified TU Electric of errors in the CPSES Unit 2 emergency core cooling system (ECCS) calculation on July 1, 1991. The peak fuel cladding temperature (PCT) for CPSES Unit 2 was calculated using the 1981 version of the Westinghouse ECCS evaluation model as identified in the letter logged TXX-6444, Reference 1, and Supplemental Safety Evaluation Reports 21 and 24 to NUREG 0797. Each change/error is identified in an attachment to this letter along with the resultant change in peak cladding temperature.

The absolute magnitude of these changes total greater than 50°F and in accordance with 10CFR50.46(a)(3)(i), have been classified as significant. This letter is TU Electric's 30 day report of changes or errors discovered in the ECCS calculations of PCT as required by 10CFR50.46(a)(3)(ii). Therefore, the information required by 10CFR50.46 is provided below.

Utilizing the attached list of changes to the Unit 2 PCT conservatively, the maximum updated value for the Unit 2 PCT is 1881°F. This updated value remains well below the 2200°F limiting value established by Part 50.46(b)(1). Therefore, the current analysis continues to demonstrate CPSES Unit 2 compliance with 10CFR50.46.

9108020152 910731  
PDR ADOCK 05000446  
PDR

400 North Olive Street L.B. 81 Dallas, Texas 75201

D029

TXX-91270  
Page 2 of 2

Furthermore, TU Electric is applying Advanced Nuclear Fuels methodology to perform a Large Break LOCA analysis for CPSES Unit 2, Cycle 1. The schedule for completion and submittal of the CPSES Unit 2, Cycle 1 analysis is August 1, 1992.

If you have any questions, please contact Mr. J. D. Seawright at (214) 812-4375.

Sincerely,

A handwritten signature in dark ink, appearing to read "William J. Cahill, Jr.", with a stylized, cursive script.

William J. Cahill, Jr.

JDS/grp  
Attachment

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (2)  
Mr. M. B. Fields, NRR

CPSES Unit 2  
Peak Cladding Temperature  
Changes/Errors

<u>Item</u>	<u>Change in PCT (°F)</u>
Allowance for WREFLOOD errors per NRC Generic Letter 86-16 (6 to 12°F)	12
Steam generator tube collapse due to concurrent seismic and LOCA loads	20
Fuel Rod Initial Condition Inconsistency	41
	<hr/>
Total Change	73
Base Calculation (1981 version of the Westinghouse ECCS evaluation model)	1808°F
Total Peak Clad Temperature	1881°F