

Georgia Power Company
Post Office Box 439
Bakley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Edwin I. Hatch Nuclear Plant

February 27, 1992

PLANT E. I. HATCH
NRC Exam Questions
Rtype: A04.43
Log: LR-GM-038-0292

Mr. Thomas A. Peebles
U. S. Nuclear Regulatory Commission
Region II, Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

Dear Mr. Peebles:

As previously discussed with Jim Moorman, our comments on the written examination administered February 24, 1992, at Plant Hatch are attached.

If you have any further questions, please call Mr. Curtis Coggin at (912) 367-7851, extension 2896.

H. L. Sumner, Jr.
General Manager Nuclear Plant

PLANT E. I. HATCH
FEB. 24, 1992
NRC EXAM COMMENTS

Question #83 RD, #82 SRO

The question asks candidate to "define" the Heat Capacity Temperature Limit. The answer which is expected is to prevent exceeding the suppression pool design temperature. This is discussed in the SRG, but is not the basis for Plant Hatch. Hatch's limit is based on the highest temperature which would not result in exceeding primary containment design pressure limit before the rate of energy transfer from the RPV is within the capacity of the containment vent.

Recommendation

Delete question due to there being no correct answer.

Question #84 RD, #89 SRO

The question asks the basis of the drywell spray initiation limit curve. The key requires "C" which is not totally correct. If I spray at high pressure I stay within the capacity of drywell/torus vacuum breakers. If I spray at low pressures then I prevent opening the torus/Rx building vacuum breakers.

Recommendation

Delete question due to no correct answer

ENCLOSURE 4

SIMULATOR FIDELITY REPORT

Facility Licensee: Georgia Power Company
Facility Docket Nos.: 50-321 and 50-366
Operating Tests Administered On: February 25 -26, 1992

This form is to be used only to report observations. These observations do not constitute, in and of themselves, audit or inspection findings and are not, without further verification and review, indicative of non-compliance with 10 CFR 55.45(b). These observations do not affect NRC certification or approval of the simulation facility other than to provide information which may be used in future evaluations. No licensee action is required solely in response to these observations.

During the conduct of the simulator portion of the operating tests, the following items were observed :

DESCRIPTION

The interlock between RHR valves F006 and F028 is not properly simulated in the model for some conditions. This interlock is required to be jumpered out when it is desired to lower Reactor Water level using the RHR system. When the jumper is installed, the F028 valve should open regardless of the position of its' handswitch. This did not occur.

The Diesel Generator battery ground detection lights constantly indicated that all buses were grounded (ie the lights were continually bright) when they were not.

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