

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

DOCKET NUMBER (2)

05000013612

PAGE (3)

1 OF 012

TITLE (4)

REACTOR TRIP BREAKER SERIAL NO. 29 UNDERVOLTAGE DEVICE ANOMALY

EVENT DATE (5)
MONTH DAY YEAR
04 28 84
LER NUMBER (6)
YEAR MONTH DAY
84 01 16
REPORT DATE (7)
MONTH DAY YEAR
05 23 84
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
DOCKET NUMBER(S)
05000013612OPERATING MODE (9)
1
POWER LEVEL (10)
1100
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)
20.402(b) 20.405(c) 50.73(a)(2)(iv) 73.71(b)
20.405(a)(1)(i) 50.36(c)(1) 50.73(a)(2)(v) 73.71(c)
20.405(a)(1)(ii) 50.36(c)(2) 50.73(a)(2)(vii) X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii) 50.73(a)(2)(i) 50.73(a)(2)(viii)(A) Informational Report
20.405(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B)
20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(x)

LICENSEE CONTACT FOR THIS LER (12)

NAME
J. G. HAYNES, STATION MANAGER
TELEPHONE NUMBER
AREA CODE
714 492-1770

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 3 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

On April 28, 1984, with Unit 3 in Mode 1 at 100% power and surveillance testing in progress, the UV trip device for RTB Serial No. 256A4002-656-29 exhibited a procedurally unacceptable response time. The breaker was left in the open position pending investigation.

The breaker's measured trip shaft torque exceeded the acceptance criterion. Following maintenance, the trip shaft torque was found to be within the acceptance criterion. Post-maintenance surveillance of the breaker yielded acceptable UV response times and the breaker was returned to service in the cross-tie position (RTB #9) on May 8, 1984.

Public health and safety were not affected since the breaker was located in the nonsafety-related cross-tie position which does not open on a reactor trip. Furthermore, the breaker continues to function properly using the shunt trip device.

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PDR ADOCK 05000362
S PDR

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQ. NUMBER	REV. NUMBER			
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	0 5 0 0 0 3 6 2	8 4	- 0 1 6	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) (EIIIS Component Code 52) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 3 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

On April 28, 1984, with Unit 3 in Mode 1 at 100% power, surveillance testing in accordance with S023-II-11.161, "Reactor Breaker Undervoltage Response Time Testing," prior to the scheduled RTB maintenance, was in progress. During this surveillance, the UV trip device for RTB #9 (Serial No. 256A4002-656-29) exhibited a procedurally unacceptable response time. The response times (in order) were 178 msec, 56 msec and 42 msec. S023-II-11.161 contains an acceptance criterion of 82 msec, which was developed from baseline testing and consideration of the Combustion Engineering (CE) guideline of 100 msec. The shunt trip feature operated properly. No Action Statements were entered since the RTB was located in the nonsafety-related cross-tie position and does not open on a reactor trip. The breaker was left open, pending investigation of the timing anomaly.

The breaker was removed from its cubicle on May 8, 1984 and maintenance was performed in accordance with S023-I-4.66, "General Electric AK-2-25 Circuit Breaker Maintenance." During this maintenance, the as-found trip shaft torques were found to be 1.78 in-lbs, 1.70 in-lbs and 1.83 in-lbs. All values exceeded the acceptance criterion of 1.5 in-lbs and could have been responsible for the observed timing anomaly. All post-maintenance trip shaft torques were found to be acceptable and were 1.27 in-lbs, 1.28 in-lbs and 1.17 in-lbs, respectively. Post-maintenance surveillance in accordance with S023-II-11.161 yielded UV response times of 37 msec, 35 msec and 35 msec. Based on this successful post-maintenance surveillance, the RTB was returned to service in the nonsafety-related cross-tie position on May 8, 1984. Future performance of the breaker will be closely monitored.

Public health and safety were not affected since the breaker was located in the nonsafety-related cross-tie position which does not open on a reactor trip. Furthermore, the breaker continues to function properly using the shunt trip device.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

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J. G. HAYNES
STATION MANAGER

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May 23, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
Informational Report
Licensee Event Report No. 84-016
San Onofre Nuclear Generating Station, Unit 3

This submittal provides an informational Licensee Event Report (LER) for an occurrence involving the Plant Protection System (PPS). The health and safety of plant personnel or the public were not affected by this occurrence.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER 84-016

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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