



NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

June 23, 1978

Mr J G Keppler
Office of Inspection & Enforcement
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Mr Keppler:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Trip of Essential Motor Control Center 133A
Supply Breaker 52-304

The Licensee Event Report for this occurrence is reproduced on
the back of this letter. Enclosed are three copies.

This event is reported in compliance with Technical Specification
6.7.B.2.b.

Yours very truly,

L O Mayer, PE
Manager of Nuclear Support Services

LOM/MHV/deh

cc: Director, IE, USNRC (30)
Director, MIPC, USNRC (3)
MPCA
Attn: J W Ferman

781810126

9105150391 780623
PDR ADOCK 05000263
S PDR

-over-

A002
5/11/78
sent to
OAC

LICENSEE EVENT REPORT

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 M N M N P 1 7 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 6 3 7 0 5 2 5 7 8 6 0 6 2 3 7 8 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 DURING NORMAL OPERATION A TRIP OF ESSENTIAL MCC B33A SUPPLY BREAKER 52-304
0 3 RESULTED IN OPERATION IN A DEGRADED MODE PERMITTED BY VARIOUS LIMITING
0 4 CONDITIONS FOR OPERATION. SYSTEMS DEGRADED WERE CORE SPRAY (T.S. 3.5.A.2), ONE
0 5 RWCU ISOLATION VALVE (T.S. 3.7.D.3), SBLC (T.S. 3.4.B.1), SBTGS (T.S. 3.7.B.1a) AND
0 6 ONE EMERGENCY SERVICE WATER PUMP. NO SIGNIFICANT OCCURRENCE TOOK PLACE AS A RESULT
0 7 OF THE EVENT. NO EFFECT ON PUBLIC HEALTH OR SAFETY. REDUNDANT PORTIONS OF ALL
0 8 SYSTEMS MENTIONED WERE AVAILABLE AND OPERABLE. NO PREVIOUS EVENT. SEE ATTACHMENT.
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 9 SYSTEM CODE E B 1 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE Z Z Z Z Z Z Z 14 COMP SUBCODE Z 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

11 REPORT NUMBER 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

ACTION TAKEN X 18 X 19 EFFECT ON PLANT 2 20 SHUTDOWN METHOD 2 21 HOURS 0 0 10 0 22 ATTACHMENT SUBMITTED Y 23 NPD-4 FORM SUB N 24 PRIME COR SUPPLIER Z 25 COMPONENT MANUFACTURER Z 9 9 9 9 26
31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 CAUSE OF OCCURRENCE UNKNOWN AT THIS TIME. EVALUATION IS CONTINUING. NUMEROUS
1 1 CHECKS HAVE BEEN MADE INVOLVING THE BREAKER, THE BREAKER CUBICLE, THE MOTOR CONTROL
1 2 CENTER BUS AND THE CONNECTED LOADS. BREAKER REPLACED WITH SPARE. INSTRUMENTATION
1 3 HAS BEEN ATTACHED TO MONITOR CURRENT SUPPLIED TO THE BUS. INVESTIGATION OF THE
1 4 POSSIBLE EFFECTS OF VARIOUS LOADING COMBINATIONS IS BEING PERFORMED.
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 5 FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION OPERATOR OBSERVATION 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 6 ACTIVITY CONTENT 2 33 2 34 NA AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER GARY R. SMITH

612/205-5151

Attachment
LER #78-009/03L-o
Northern States Power Company
Monticello Nuclear Generating Plant
Docket No. 50-265

During the followup investigation of the initial trip, on June 2, 1978, the replacement breaker tripped. The operator placed the mode switch in SHUTDOWN causing a reactor scram. Following the scram the tripped breaker was reclosed and operated properly. During the shutdown the breaker tripped a third time. The breaker was reclosed and no further trips have occurred to date. System degradation during these two trips was identical to the original event.

L06/30/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

56-263

REC: KEPPLER J G
NRC

ORG: MAYER L O
H STATES PWR

DOCDATE: 06/23/78
DATE RCVD: 06/30/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-263/78-009) ON 05/25/78 CONCERNING TRIP
OF ESSENTIAL MCC B33A SUPPLY BREAKER 52-304 RESULTED IN OPERATION IN A
DEGRADED MODE PERMITTED VARIOUS LIMITING CONDITIONS FOR OPERATION. W/ATT.

PLANT NAME: MONTICELLO

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: K

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORT
(DISTRIBUTION CODE A002)

FOR ACTION: DR OFF GR#3 BC**W/4 ENCL

INTERNAL: REG FILE**W/ENCL
E**W/2 ENCL
I & C SYSTEMS DR**W/ENCL
NOVAK/CHECK**W/ENCL
AD FOR ENG**W/ENCL
HANAUER**W/ENCL
AD FOR SY'S & P&ID**W/ENCL
ENGINEERING BR**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC LDR**W/ENCL
MIPC**W/3 ENCL
EMERGENCY PLAN BR**W/ENCL
EEB**W/ENCL
PLANT SYSTEMS BR**W/ENCL
AD FOR PLANT SYSTEMS**W/ENCL
REACTOR SAFETY BR**W/ENCL
VOLLMER/BUNCH**W/ENCL
POWER SYS BR**W/ENCL

EXTERNAL: LFDR'S
MINNEAPOLIS, MN* W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+1P

CONTROL NBR: 791910126

***** THE END *****