

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3										DOCKET NUMBER (2) 0 5 0 0 0 3 6 2				PAGE (3) 1 OF 0 1		
TITLE (4) HIGH STEAM GENERATOR WATER LEVEL REACTOR TRIPS																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 5	0 5	8 4	8 4	0 1 7	0 0	0 5	2 1	8 4					0 5 0 0 0 1 1			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
2		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)				
POWER LEVEL (10)		0 0 0				20.405(a)(1)(i)				50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER AREA CODE 7 1 4 4 9 2 - 7 7 0 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 5, 1984, with Unit 3 in Mode 2 at .3 percent power, and on May 8, 1984, with Unit 3 in Mode 1 at 8 percent power, a reactor trip occurred on high steam generator water level. Both trips resulted from overfeeding the steam generators with the Feedwater Control System (EIIIS System Code JB) in manual control during routine reactor shutdowns. All plant systems responded normally to stabilize plant conditions during these events.

Manual steam generator level control is difficult at low power due to the "shrink" and "swell" responses of steam generator water level. As previously reported in LER 84-020 (Docket No. 50-361), design changes to optimize steam generator water level control at all power levels are under consideration.

There are no reasonable or credible alternative circumstances under which this event would have been more severe.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

SCE

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

May 21, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-017
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73.a.2(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the actuation of the Reactor Protection System. The health and safety of the public or plant personnel were not affected by this event.

If you require any additional information, please so advise.

Sincerely,

J. G. Haynes

Enclosure: LER No. 84-017

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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