

NSP

Regulatory

File Of.

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

April 1, 1974

Mr. J F O'Leary
Directorate of Licensing
Office of Regulation
U S Atomic Energy Commission
Washington, DC 20545



Dear Mr. O'Leary:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. JO-263 License No. DPR-22

Supplemental Information to the Monticello Second
Reload Submittal

On November 19, 1973 we transmitted a report entitled "Monticello Nuclear Generating Plant Second Reload Submittal". Figure 2-a was inadvertently omitted. Please incorporate the attached replacement pages in the report(s) sent to you on November 19, 1973 to correct this omission.

Remove Old Pages

1-1/1-2
2-1/3-1

Insert New Pages

1-1/2-1
2-2/3-1

Yours very truly,

A handwritten signature in cursive script, appearing to read "L. O. Mayer".

L O Mayer, PE
Director of Nuclear Support Services

LOM/MHV/kn

cc: J G Keppler
G Charnoff
Minnesota Pollution Control Agency
Attn. E A Pryzina



attachment

2888

9105060220 740401
PDR ADDCK 05000263
P PDR

1. INTRODUCTION

This document provides the technical basis of the license submittal for the second reload of Northern States Power/Monticello Unit 1. Presented herein is a description of the new fuel and the results of the evaluation of the refueled core for the February, 1974 outage.

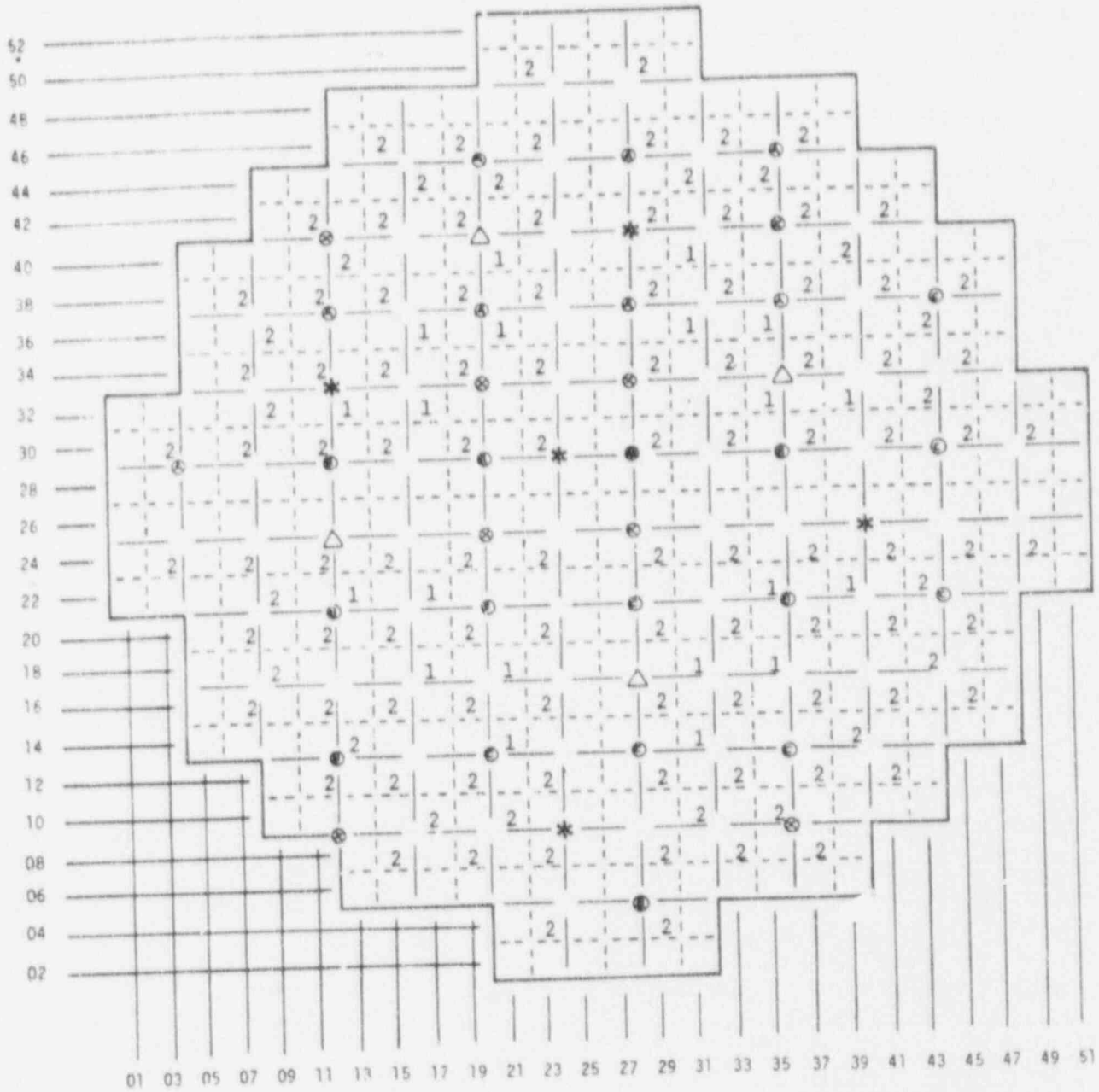
The fuel at the site available for loading at the outage will be 116 Reload-2 fuel bundles, which are 8x8 bundles with an average enrichment of 2.62 wt% of U-235. There will also be available for reinsert 7 initial core bundles discharged at the end of Cycle 1 with an average exposure of about 7400 Mwd/t.

The objective of this outage is to remove the 44 temporary control curtains which remained in the core during Cycle 2 and to load the core so as to assure the availability of the plant at high power for approximately an annual cycle.

Sections 3, 4, 5 and 6 of this document, dealing with the subjects of reload fuel mechanical design and reloaded core thermal-hydraulic and nuclear characteristics, and safety analysis, present descriptions of design criteria, methods and results from design calculations and safety evaluations and represents complete information for the review of fuel assembly and core design.



MONTICELLO



- ⊗ LPRM Location (Letter indicates TIP machine)
- LPRM Location (Common location for all TIP machines)
- ⊗ IRM Locations
- △ SRM Locations
- * Source Locations
- 1 : Reload-1 Fuel
- 2 : Reload-2 Fuel

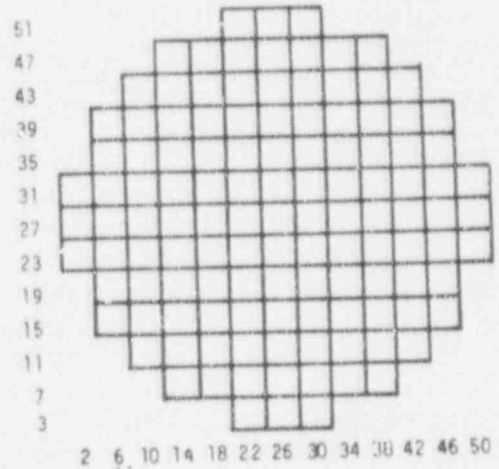


Figure 2-1. Reference Core Loading, Monticello Cycle 3

AEC DI IBUTION FOR PART 50 DOCKET MAT AL
(TEMPORARY FORM)

CONTROL NO: 2888

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TO: J. F. O'Leary			ORIG	CC 40	OTHER	SENT AEC PDR <u>xxx</u> SENT LOCAL PDR <u>xxx</u>		
CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D 40		DOCKET NO: 50-263		
	XXXX							

DESCRIPTION:

Ltr trans the following...

ACKNOWLEDGED

PLANT NAME: MONTICELLO

ENCLOSURES:

Fig 2-1 to report "Monticello Nuclear Generating Plant Second Reload Submittal" inadvertently omitted from original submittal

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FOR ACTION/INFORMATION

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