

GPU Nuclear Corporation

One Upper Pond Road
Parsippany, New Jersey 07054
201-316-7000
TELEX 136-482
Writer's Direct Dial Number

March 17, 1992

C321-92-2056

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Facility Operating License Amendment Request No. 207

Pursuant to 10 CFR 50.90, GPU Nuclear Corporation, operator of the Oyster Creek Nuclear Generating Station (OCNGS), Facility Operating License No. DPR-16, requests a change to that license.

Technical Specification (TS) 4.2.E.5 requires a B-10 enrichment surveillance of the Standby Liquid Control System at each refueling outage with analyses available within 30 days after startup. Since OCNGS procures the sodium pentaborate pre-enriched (B-10) and the B-10 is very stable, this TS unduly restricts the performance of this surveillance to a plant shutdown. In order to allow flexibility in the master surveillance schedule, OCNGS proposes to change the surveillance interval to once every 24 months.

This change request has been reviewed in accordance with Section 6.5 of the OCNGS Technical Specifications, and using the standards in 10 CFR 50.92 we have concluded that this proposed change does not constitute a significant hazards consideration.

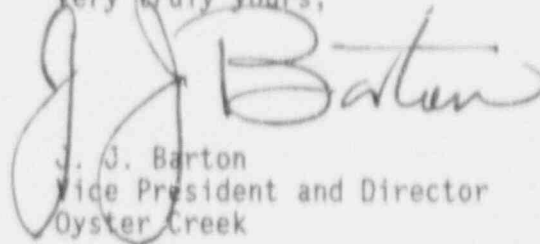
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Pursuant to 10 CFR 50.91(b)(1), a copy of this change request has been sent to the State of New Jersey Department of Environmental Protection.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. J. Barton". The signature is fluid and cursive, with the first name "J. J." and the last name "Barton" clearly distinguishable.

J. J. Barton
Vice President and Director
Oyster Creek

Attachments
JJJ/DGJ/plp (OTSCR207.LET)

cc: Administrator, Region I
NRC Resident Inspector
Oyster Creek NRC Project Manager



GPU Nuclear Corporation

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Parsippany, New Jersey 07054
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March 17, 1992

Mr. Kent Tosch, Director
Bureau of Nuclear Engineering
Department of Environmental Protection
CN 411
Tranton, NJ 08625

Dear Mr. Tosch:

Subject: Oyster Creek Nuclear Generating Station
Facility Operating License No. DPR-16
Facility Operating License Amendment Request No. 207

Pursuant to 10 CFR 50.91(b)(1), please find enclosed a copy of the subject document which was filed with the United States Nuclear Regulatory Commission on March 17, 1992.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. J. Barton".

J. J. Barton
Vice President and Director
Oyster Creek

Attachment
JJB/DGJ/plp



GPU Nuclear Corporation

One Upper Pond Road
Parsippany, New Jersey 07054
201-316-7000
TELEX 136-482
Writer's Direct Dial Number

March 17, 1992

The Honorable Russell C. Palumbo
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Dear Mayor Palumbo:

Enclosed herewith is one copy of Facility Operating License Amendment Request No. 207 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on March 17, 1992.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. J. Barton".

J. J. Barton
Vice President and Director
Oyster Creek

Attachment
JJB/DGJ/plp

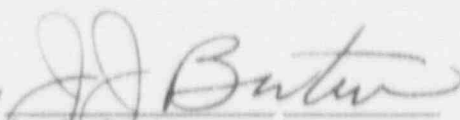
GPU NUCLEAR CORPORATION
OYSTER CREEK NUCLEAR GENERATING STATION

Facility Operating
License No. DPR-16

Facility Operating License Amendment
Request No. 207
Docket No. 50-219

Applicant submits, by this Facility Operating License Amendment Request No. 207 to the Oyster Creek Nuclear Generating Station Operating License, a change to pages 4.2-2 and 4.2-4.

BY


J. J. Barton
Vice President and Director
Oyster Creek

Sworn and Subscribed to before me this 17th day of March 1992.


Notary Public of NJ

JUDITH M. CROWE
Notary Public of New Jersey
My Commission Expires 12/31/95

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of
GPU Nuclear Corporation

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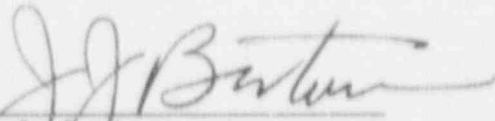
Docket No. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Facility Operating License Amendment Request No. 207 for Oyster Creek Nuclear Generating Station Operating License, filed with the U.S. Nuclear Regulatory Commission on March 17, 1992 has this day of March 17, 1992, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Russell C. Palumbo
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

By


J. J. Barton
Vice President and Director
Oyster Creek

OYSTER CREEK NUCLEAR GENERATING STATION
OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
FACILITY OPERATING LICENSE AMENDMENT REQUEST NO. 207

Applicant hereby requests the Commission to change Facility Operating License DPR-16 as discussed below, and pursuant to 10 CFR 50.91, an assurance concerning the determination of no significant hazards also presented:

TO BE CHANGED

1 Basis

2.0 EXTENSION CHANGE

Change the surveillance for Boron-10 enrichment to reflect the use of sodium pentaborate preformulated and pre-enriched at the vendor's facility.

3.0 CHANGES REQUESTED

As delineated on the attached revised Technical Specification pages 4.2-2 and 4.2-3.

4.0 DISCUSSIONS

Paragraph (c)(4) of 10 CFR 50.62, "Requirements for reduction of risk from Anticipated Transients Without Scram (ATWS) events for light-water-cooled nuclear power plants", requires that each boiling water reactor must have a Standby Liquid Control System (SLCS) with a minimum flow capacity and boron content equivalent in control capacity to 86 gallons per minute of 13 weight percent sodium pentaborate solution. OCNGS meets the equivalency requirements based on a minimum solution concentration of 15.0 wt. percent, a Boron-10 enrichment of 35 atom percent and a 30 gpm pump flow rate.

The approach taken by OCNGS was one of three options approved by the NRC staff as an acceptable response to the ATWS Rule (Reference: Safety Evaluation of Topical Report NEDE-31096-P, "Anticipated Transient Without Scram; Response to ATWE Rule 10 CFR 50.62," transmitted by letter, G. Lainas, NRC, to T. Pickens, BWR Owners Group, dated October 21, 1986). Associated with this option was a requirement for surveillance and positive verification of the correct Boron-10 enrichment. These surveillances were discussed with the NRC staff during a BWR Owners' Group meeting on April 3, 1987.

4.0 DISCUSSIONS (Continued)

OCNGS purchases sodium pentaborate preformulated and pre-enriched at the vendor's facility. The B-10 enriched sodium pentaborate is procured in accordance with the vendor's quality assurance plan as approved by GPU Nuclear. Samples of batches to be shipped are analyzed prior to shipment by an independent laboratory contracted by GPU Nuclear. Once verification is completed it is only then that these batches are shipped to the plant site. The chemical, while in storage or in the SLCS tank, cannot change its B-10 enrichment, because B-10 is very stable. Since chemicals used in making up the solution in the SLCS tank are only from these batches, there is no reason why the resulting solution will be below the minimum B-10 enrichment. Water addition to the tank can change the weight percent of the sodium pentaborate solution but cannot change the B-10 enrichment.

The current Technical Specification 4.2.E.5 requires the performance of the B-10 enrichment surveillance at each refueling outage with the receipt of the analysis no later than 30 days after startup. The intent of this surveillance is to provide a periodic verification, at least once per operating cycle, of the B-10 enrichment as found in the SLCS tank.

Since the samples for the enrichment analysis can be taken from the SLCS tank at any time (power operation, refueling, etc), the current Technical Specification unduly restricts the performance of this surveillance to plant shutdowns. Further, the B-10 is very stable which precludes a change while in the SLCS tank. Since the intent is to verify the B-10 enrichment once per operating cycle, there is no specific need to restrict this surveillance to only a refueling outage when the surveillance can be performed at any time. Based on these facts, OCNGS proposes to change the surveillance to once every 24 months. This change will retain a defined schedule for the surveillance, but the change will allow flexibility in performing the surveillance.

5.0 DETERMINATION

GPU Nuclear has determined that operation of the Oyster Creek Nuclear Generating Station in accordance with the proposed Technical Specifications does not involve a significant hazard. The changes do not:

1. Involve a significant increase in the probability or the consequence of an accident previously evaluated.

The proposed changes reflect the use of preformulated and pre-enriched sodium pentaborate in the SLCS. Since the sodium pentaborate is independently verified for the B-10 enrichment prior to shipment, an enrichment analysis of the SLCS tank once every twenty-four months is deemed adequate to ensure that the reactor can be brought to a cold shutdown condition from full power steady state operating conditions at any time

in core life independent of control rod system capabilities. The probability or the consequences of an accident previously evaluated are unaltered by the proposed change.

2. Create the possibility of a new or different kind of accident from any previously evaluated.

The proposed changes do not modify the system design for the SLCS nor the manner in which the SLCS would be operated. Since the SLCS retains the capability to shutdown the reactor, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed changes allows the performance of the B-10 enrichment surveillance during power operation or plant shutdown. Since the frequency of the surveillance stays the same, once per twenty-four months, there is no significant reduction in the margin of safety. The SLCS retains the capability to bring the reactor to a cold shutdown condition from full power steady state operating conditions at any time in core life independent of control rod system capabilities.

6.0 IMPLEMENTATION

It is requested that the amendment authorizing this change become effective 60 days after issuance.