

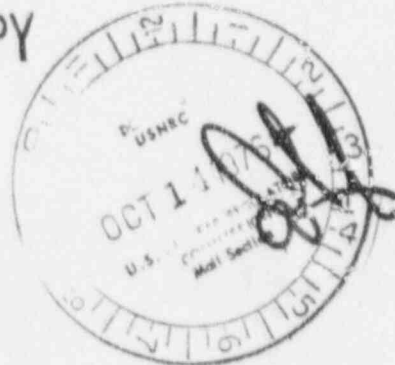
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NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

October 12, 1976 **REGULATORY DIVISION FILE COPY**

Mr Victor Stello, Director
Division of Operating Reactors
U S Nuclear Regulatory Commission
Washington, DC 20555



Dear Mr Stello:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

- References:
- (a) NUTECH Report NSP-01-140, "Monticello Nuclear Generating Plant Short Term Program Plant Unique Torus Support and Attached Piping Analysis", August, 1976
 - (b) GE Report No. NEDC-20989-P, "Mark I Containment Evaluation Short Term Program, Addendum 2, Loads and Their Application for Torus Support System Evaluation", June, 1976
 - (c) GE Report No. NEDC-20989-P, Addendum 3, "Mark I Containment Evaluation Short Term Program Final Report", August, 1976

Mark I Containment Information

Attached are forty copies of NUTECH Report NSP-01-168 titled "Monticello Nuclear Generating Plant Supplement to Short Term Program Plant Unique Torus Support and Attached Piping Analysis". This report presents an evaluation of the effects of maximum torus water level on STP loads, as reported in reference (a); and an evaluation of the effects of input parameter errors on the results reported in reference (a). Appendix A of the attached report contains revised tables to references (b) and (c), which show the corrected input data and analytical results.

A change to reference (a) is noted in Appendix B of the attached report. Reference 10 of reference (a) should be changed to read as shown above in reference (c).



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NORTHERN STATES POWER COMPANY

Mr Victor Stello
Page 2
October 12, 1976

Responses to the NRC staff questions of January, 1976, on NEDC-20989, were submitted by a letter dated September 9, 1976 from L J Sobon to V Stello. We have found the information reported in the September 9 letter to be applicable to Monticello.

Several of the January, 1976 staff questions concerning plant unique information were not addressed in the September 9 letter. Responses to the plant unique questions are as follows:

Structural Engineering Branch

4. Short term repairs were reported in a letter to Mr Stello dated May 10, 1976.
5. As a result of the short term repairs reported on May 10, 1976, there are no critical structural elements on the Monticello Containment that are near yield due to pool swell loads. A schedule of inspections of pin joints and anchorage will be instituted to ensure their continued integrity. Mechanical fasteners on the catwalks, vent header support pin joints and safety relief valve discharge piping supports inside the torus will be visually inspected each time the torus is drained. Inspections of the pin joints and anchorage on the torus support system and ECCS header will be visually inspected during the course of normal plant inspections. Previous detailed inspections of these systems have not uncovered any deficiencies and, therefore, are supportive of the inspection intervals stated above.
7. Answer contained in reference (a).
8. Not applicable to Monticello.
10. Answer contained in reference (a).
30. Same as answer to 5.

Containment Systems Branch

8. The instrument airlines inside the Monticello torus are those going to the vacuum breakers. There is only one containment penetration for the air supply, which has a normally shut isolation valve outside of containment. The air supply is used only for periodic surveillance testing of the vacuum breakers and is not required for them to perform their accident function. A screening analysis of this arrangement was presented in the STP Report, Volume III, Section 4.3.8.
9. Not applicable to Monticello.
19. Answer contained in reference (a).
20. Answer contained in reference (a).

NORTHERN STATES POWER COMPANY

Mr Victor Stello

Page 3

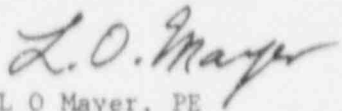
October 12, 1976

Mechanical Engineering Branch

3. Answer contained in reference (a).
4. Answer contained in reference (a).
7. Answer contained in a letter to Mr D L Ziemann dated December 16, 1975.

Responses to the NRC staff questions of August, 1976 related to the Long Term Program content were submitted by a letter from Mr L J Sobon to V Stello dated August 25, 1976. We have found the information in Mr Sobon's letter to be applicable to Monticello.

Yours very truly,



L O Mayer, PE
Manager of Nuclear Support Services

LOM/LLT/ak

cc: J G Keppler
G Charnoff
MPCA
Attn: J W Ferman

Attachment

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

Docket No. 50-263

License No. DPR-22

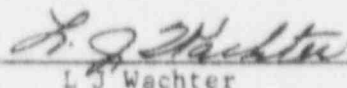
LETTER DATED October 12, 1976
RESPONDING TO NRC REQUESTS
FOR INFORMATION ON CONTAINMENT DESIGN

Northern States Power Company, a Minnesota corporation, by this letter dated October 12, 1976 hereby submits information in response to NRC requests for information concerning the Mark I Containment.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By



L J Wachter

Vice President, Power Production
& System Operation

On this 12th day of October 1976, before me a notary public in and for said County, personally appeared L J Wachter, Vice President, Power Production and System Operation, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.



