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JUL 24 1983

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

August 19, 1983
ANPP-27593-RQT/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-49
A 50.55(e) Potentially Reportable Deficiency Relating
to Diffuser Bolts Broken/Loose In Reactor Coolant Pump
File: 83-019-026
D.4.33.2

Reference: Telephone Conversation between P. Narbut and R. Tucker
on July 15, 1983

Dear Sir:

The NRC was notified of a Potentially Reportable Deficiency in the
referenced telephone conversation. At that time, it was estimated
that a determination of reportability would be made within thirty
(30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by November 29, 1983, at which time a complete report will
be submitted.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/RQT:skc

Attachment

cc: See Attached Page 2

8405250289 840423
PDR FOIA
BERNABE84-212 PDR

#38

U. S. Nuclear Regulatory Commission
Attention: Mr. D. M. Sternberg, Chief
ANPP-27593-RQT/BSK
August 19, 1983
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
J. A. Roedel
D. B. Fasnacht
A. C. Rogers
B. S. Kaplan
W. E. Ide
J. Vorees
J. R. Bynum
P. P. Klute/D. D. Green
A. C. Gehr
W. J. Stubblefield
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R. W. Welcher
R. M. Grant
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G. A. Fiorelli

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

J. G. Self
S. R. Frost

INTERIM REPORT - DER 83-49
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT #1

I. Potential Problem

Post inspection of Unit #1 Reactor Coolant Pump, Tag Number 1MRCEP01A, revealed the following damage occurred during hot functional testing:

- A. Fifty percent (50%) of all diffuser bolts were loose.
- B. Three (3) diffuser bolts were broken and the pieces are missing.
- C. The diffuser and suction pipe areas were damaged.

II. Approach To and Status Of Proposed Resolution

Bechtel Engineering is corresponding with CE to provide reportability disposition and technical justification for corrective action.

III. Projected Completion of Corrective Action and Submittal of the Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by November 29, 1983.

DATE _____

Facility

Notification

Item or Event

Regional Action

DER 83-46

B. During a hydrostatic test a Borg Warner Valve was manually closed resulting in failure of $5/16$ inch bolts which mount the actuator to the yoke. The cause of failure is not known at this time although excessive force is suspected.

The licensee will provide a written report in 30 days if the items are determined to be reportable.

DATE _____

Facility	Notification	Item or Event	Regional Action
Palo Verde Palo Verde Electric Service Company DN 50-523 529 530	Telephone call from licensee 7/15/83 A.	Potential Construction Deficiency Reports Reactor Coolant Pump 1A diffuser ring cap screws failed. Five of sixteen failed, three had pieces of the bolt missing. The diffusers and the suction pipe are damaged. The pump was being disassembled for other reasons when the damage was discovered. Note that these bolts were the subject of a previous report ^(DER 82-61) when the same bolts failed in the CE-KSB test loop. The ^{CE} required modifications had been made at Palo Verde. The previous test loop event was reported as a potential generic item 6/3/83.	Followup per MC 2512

DER 83-49

J. Blair
- PPN
HMS

MORNING REPORT - REGION V
DATE: 08/02/83

FACILITY	NOTIFICATION	ITEM OR EVENT	REGIONAL ACTION
Arizona Public Service Co. Palo Verde 1,2,3 DN's 50-528 50-529, 50-530	Telephone call from licensee 8/1/83	POTENTIAL CONSTRUCTION DEFICIENCIES: <ul style="list-style-type: none">Unit 1 Turbine Driven Auxiliary Feed Pump. The pump shaft vibrates badly at 135 gpm (minimum flow operation) causing the suction piping to vibrate severely. Additionally, the pump does not develop 1420 psig at a flow of 875 gpm as required by the hot functional test requirements. (DER 83-51)The Unit 1 125 volt vital DC power bus distribution panel experiences 70 volt AC spikes at a 0.0007 second frequency. Discovered during testing. (DER 83-52)Unit 3 containment embedment plates failed to meet specification requirements. Two plates were discovered without anchor bolts installed. Nine plates, accepted by inspection, did not meet minimum thread engagement requirements for the anchor bolts. Approximately 300 other plates, not yet final accepted, also lacked anchor bolt thread engagement. (DER 83-53)Inspection of the Unit 1 steam generator primary sides revealed debris. SG2 had 11 washers and one threaded plug. SG1 had one washer. (DER 83-54)The Unit 1 thermal sleeve for line SI-A207-DCCA-14 was visually determined to be missing and is expected to be found in the reactor vessel. (DER 83-55)During the conduct of hot functional testing in Unit 1, the licensee declares that continual operational difficulties were experienced with the MSIV's by Anchor Darling Company. Repeated failures required rework and retest. (DER 83-56)	Followup per MC 2512

(F)

441

The licensee will submit a written report in 30 days if the above six items are determined to be reportable.

DATE _____

Facility

Notification

Item or Event

Regional Action

Arizona Public
Service
#PUNGS-1
DA. 50-528

Telephone
call from
licensee on
7/1/83

Potentially Reportable 50.55(e):
Licensee has discovered possible
reportable indications on a Reactor
Coolant Pump during a preservice
UT ~~examination~~ examination. The
indications are in the pump casing
girth weld area. The indications had not
been identified on shop RT's performed by
C-E Chattanooga. Licensee is evaluating
and if found reportable will file a written report
within 30 days.

Follow-up
per
MC 2512

CC Norbut

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

August 22, 1983
ANPP-27604-BSK/RQT

39
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133 SEP -1 PM 1-17
REGION 1

U. S. Nuclear Regulatory Commission
Region V
Creekside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-55
A 50.55(e) Potentially Reportable Deficiency Relating to
Thermal Liner In Safety Injection System Line Is Missing.
File: 83-019-026; D.4.33.2

Reference: Telephone Conversation between P. Narbut and R. Tucker
on August 1, 1983.

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the referenced telephone conversation. At that time, it was estimated that a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim Report is attached. It is now expected that this information will be finalized by November 9, 1983, at which time a complete report will be submitted.

Very truly yours,

E. E. Van Brunt, Jr.

E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVBJr./RQT:sn

Attachment

cc: See Attached Page Two

8309120320

39

U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
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Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

INTERIM REPORT - DER 83-55
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. Potential Problem

During visual inspection of piping, after disassembly of Unit 1 pump tag no. 1-RCE-P01A, it was revealed that the thermal liner for line number SI-A207-BCCA-14 was missing. Preliminary determination is that the liner broke loose during hot function testing and was flushed into the Reactor Vessel.

II. Approach To and Status Of Proposed Resolution

Bechtel engineering is corresponding with C-E to provide reportability disposition and technical justification for corrective action.

III. Projected Completion Of Corrective Action and Submittal of The Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by November 9, 1983.

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

August 22, 1983
ANPP-27603-BSK/RQT

RECEIVED
1983 SEP -1 PM 1:17

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-54
A 50.55(e) Potentially Reportable Deficiency Relating to
Washers Found In Unit 1 Steam Generator #1 And #2
File: 83-019-026; D.4.33.2

Reference: Telephone Conversation between P. Narbut and R. Tucker on
August 2, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the referenced telephone conversation. At that time, it was estimated that a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim Report is attached. It is now expected that this information will be finalized by October 13, 1983, at which time a complete report will be submitted.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVBJr./RQT:sn

Attachment

cc: See Attached Page Two

8309120030

40

U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

INTERIM REPORT - DER 83-54
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT I

I. Potential Problem

During initial inspection of Steam Generator #2, eleven (11) bevel washers and one plug were found inside the Steam Generator on the side next to the Reactor Vessel. One (1) washer was found in Steam Generator #1.

II. Approach To and Status Of Proposed Resolution

Bechtel engineering is corresponding with C-E to provide reportability disposition and technical justification for corrective action.

III. Projected Completion Of Corrective Action and Submittal of The Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by October 13, 1983.

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

42
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1983

August 1, 1983
ANPP-27471-BSK/RQT

U. S. Nuclear Regulatory Commission
Region V
Creekside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-39
A 50.55(e) Potentially Reportable Deficiency Relating to
Rejectable Indications on CE Supplied Unit 1 Reactor Coolant
Pump 1A.
File: 83-019-026; D.4.33.2

Reference: Telephone Conversation between A. D'Angelo and R. Tucker on
July 1, 1983.

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the
referenced telephone conversation. At that time, it was estimated that a
determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by October 27, 1983, at which time a complete report will be
submitted.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVBJr./RQT:rb
Enclosure

cc: See Page Two

8308220198

#42

U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
J. A. Roedel
D. B. Fasnacht
A. C. Rogers
B. S. Kaplan
W. E. Ide
J. Vorees
J. R. Bynum
P. P. Klute/D. D. Green
A. C. Gehr
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1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

8/1/83

INTERIM REPORT - DER 83-39
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. POTENTIAL PROBLEM

On a recent re-examination and comparison between the Preservice Examination Ultrasonic (UT) data and the construction Radiographs (RT) for the Combustion Engineering (C-E) furnished Reactor Coolant Pump 1A/IMRCEPOIA, two linear indications were found. The indications exist in the circumferential weld of the pump casing.

One of the RT indications found measures 3 inches long; the second measures 3/4 inches long. Both exist within a length of 6 inches. Subsequent examinations have shown that these indications are approximately 3.2 inches in from the outside diameter of the casing with no perceptible through wall dimension. The base metal of the casing in this area is 5-5/8 inches thick. The linear indications are acceptable to the ASME Section III UT acceptance criteria but do not meet the ASME Section III RT acceptance criteria. A Fracture Mechanics Analysis has been performed by C-E on the casing using conservative parameters. The analysis assumed: (1) the indications have a through wall dimension of 3/4 inches; and (2) the indication is a sharp crack extending around the entire circumference of the casing.

II. APPROACH TO AND STATUS OF PROPOSED RESOLUTION

Bechtel is continuing to correspond with CE on this matter to determine reportability.

III. PROJECTED COMPLETION OF CORRECTIVE ACTION AND SUBMITTAL OF THE FINAL REPORT

Evaluation of this condition and submittal of the Final Report is forecast to be completed by October 27, 1983.

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

July 25, 1983
ANPP-27398-BSK/RQT

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NRC

1983 AUG -1 PM 1:50

REGION V

U. S. Nuclear Regulatory Commission
Region V
Creekside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-42
A 50.55(e) Potentially Reportable Deficiency Relating to
Thermowells in Reactor Coolant System
File: 83-019-026; D.4.33.2

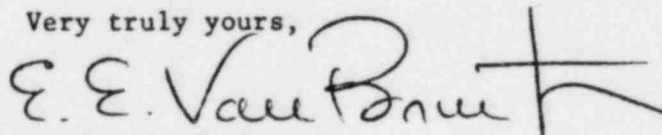
Reference: Telephone Conversation between P. Johnson and J. Roedel on
June 29, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the referenced telephone conversation. At that time, it was estimated that a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim Report is attached. It is now expected that this information will be finalized by September 21, 1983, at which time a complete report will be submitted.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVBJr./RQT:rb
Enclosure

cc: See Page Two

8308030247

#43

U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
J. A. Roedel
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1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

INTERIM REPORT - DER 83-42
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. Potential Problem

During hot functional testing, it was discovered that thermowells 01JRCA-TW-122CA and 01JRCC-TW-112CC were not maintaining the pressure boundary in the Reactor Coolant System and were emitting steam through their respective temperature elements, tag numbers 01JRCA-TE-122CA and 01JRCC-TE-112CC. Preliminary investigation reveals penetration cracks on the thermowells internal wall surface. Some damage to temperature elements was also observed.

II. Approach To and Status of Proposed Resolution

Bechtel engineering is corresponding with C-E to provide reportability disposition and technical justification for corrective action.

III. Projected Completion of Corrective Action and Submittal of the Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by September 21, 1983.

(44)

RECEIVED
1983 AUG -1 PM 1:10
REGIO. 11

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

July 25, 1983
ANPP-27396-BSK/RQT

U. S. Nuclear Regulatory Commission
Region V
Creekside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-40
A 50.55(e) Potentially Reportable Deficiency Relating to LPSI
And CS Pump Motors Leaking Oil And Have Defective Wells
File: 83-019-026
D.4.33.2

Reference: Telephone Conversation between A. D'Angelo and R. Tucker
on June 29, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the referenced telephone conversation. At that time, it was estimated that a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim Report is attached. It is now expected that this information will be finalized by October 6, 1983, at which time a complete report will be submitted.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVBJr./RQT:rb
Enclosure

cc: See Page Two

8308050376

#44

U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
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J. Self

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

INTERIM REPORT - DER 83-40
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. Potential Problem

The LPSI and CS Pump Motors "A" and "B" are discrepant as follows:

1. Lower bearing leaking oil.
2. Defective welds on the lower motor end brackets.

II. Approach To and Status Of Proposed Resolution

Bechtel engineering is corresponding with C-E, to provide reportability disposition, and technical justification for corrective action.

III. Projected Completion of Corrective Action and Submittal of the Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by October 6, 1983.

DEI
83-21

FOR: EXECUTIVE DIRECTOR FOR OPERATIONS

DAILY REPORT REGION V
DATE: ~~2/25/83~~ 2/6/83

Regional Action

Follow-up per

MC2512

Facility

Arizona
Public
Service
PVNGS-1, 2
DN# 50-528,
~~50-529~~

Notification

Telephone call
from licensee
on ~~2/25/83~~

2/5/83

Item or Event

Potentially Reportable 50.55(e): A Low Pressure Safety Injection (LPSI) pump was operated in violation of procedure and possibly damaging the pump. Pump was operated with suction valve closed. Licensee is evaluating and if found reportable will file a written report within 30 days.

Ops resp

DER
83-40

DAILY REPORT REGION V

DATE _____

Facility

Notification

Item or Event

Regional Action

Arizona Public
Service
PVNGS-1
Dn 50-528

Telephone call
from licensee
on 6/29/83

Potentially Reportable 50.55(e);
Licensee startup group has
discovered during preoperational

Follow-up
per HC 2512

testing the LPCI and Containment spray
pump motors for A and B trains are leaking lube
oil and have defective welds in the motor end
brackets. The motors were manufactured by
Westinghouse and supplied by C-E the NSSS
supplier. Licensee is evaluating and if found
reportable will file a written report within 30 days.

cc Norbut

45

RECEIVED
NRC
1983 JUL 27 PM 12:55
REGIONAL

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

July 18, 1983
ANPP-27333-RQT/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Time Extension for Report - DER 83-09
A 50.55(e) Potentially Reportable Deficiency Relating to Unit 1
LPSI Pump "A" Suction Strainer Contains Foreign Material, And
Pump "B" Strainer is Torn.
File: 83-019-026, D.4.33.2

Reference: (A) Telephone Conversation Between T. Young and G. Duckworth
on February 16, 1983.
(B) ANPP-23238 dated March 14, 1983 (Interim Report)

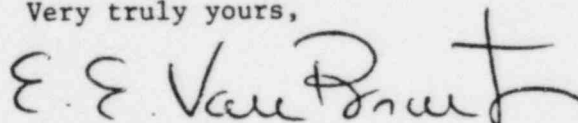
Dear Sir:

The NRC was notified of a Potentially Reportable Deficiency in Reference (A), and an Interim Report was transmitted by Reference (B). At that time, it was estimated that a Final Report would be available by July 15, 1983.

Due to the extensive investigation and evaluation required, it is now expected that this information will be finalized by September 21, 1983, at which time a complete report will be submitted.

There is no new information or change to the Interim Report at this time.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/RQT:ru

cc: See Page 2

8308030108

#45

U. S. Nuclear Regulatory Commission
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
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Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

46

MAY 12 1983

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

May 11, 1983

ANPP-23731-RQT/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Time Extension For Report - DER 83-9
A 50.55(e) Potentially Reportable Deficiency Relating to
Unit 1 LPSI Pump "A" Suction Strainer Contains Foreign
Material, and Pump "B" Strainer is Torn
File: 83-019-026; D.4.33.2

Reference: (A) Telephone Conversation between T. Young and
G. Duckworth on February 16, 1983
(B) ANPP-23238, dated March 14, 1983 (Interim Report)

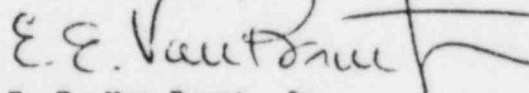
Dear Sir:

The NRC was notified of a potentially reportable deficiency in
Reference (A), and an Interim Report was transmitted by Reference (B).
At that time, it was estimated that a Final Report would be available by
May 12, 1983.

Due to the extensive investigation and evaluation required, it is now
expected that this information will be finalized by July 15, 1983, at
which time a complete report will be submitted.

There is no new information or change to the Interim Report at this time.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/RQT:db

cc: See Attached Page 2

8306010137

46

Mr. D. M. Sternberg
May 11, 1983
ANPP-23731-RQT/BSK
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
J. A. Roedel
D. B. Fasnacht
G. C. Andognini
J. R. Bynum
A. C. Rogers
B. S. Kaplan
W. E. Ide
J. Vorees
P. P. Klute/D. D. Green
A. C. Gehr
W. J. Stubblefield
W. G. Bingham
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R. W. Welcher
R. M. Grant
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G. A. Fiorelli

47
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MAY 13 1983
MAY 6, 1983
ANPP-23692-RQT/BSK

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

May 6, 1983

ANPP-23692-RQT/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Deficiency Evaluation Report **DER 83-21**
A 50.55(e) Evaluation Relating to
A LPSI Pump Was Operated In Violation Of Procedures
Possibly Causing Damage
File: 83-019-026; D.4.33.2

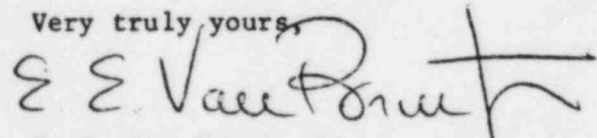
Reference: Telephone Conversation between A. D'Angelo and
R. Tucker on April 6, 1983

Dear Sir:

The NRC was notified of the subject Deficiency Evaluation in the
referenced telephone conversation, with a determination of reportability
to be made within thirty (30) days (May 5, 1983).

The deficiency has subsequently been determined to be **Not Reportable**
under the requirements of 10CFR50.55(e), therefore no written report is
required. Necessary documentation of the evaluation will be maintained
in APS files.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVB/RQT:db

Enclosure

cc: See Page 2

8305230517

47

Mr. D. M. Sternberg
May 6, 1983
ANPP-23692-RQT/BSK
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
J. A. Roedel
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L. E. Vorderbrueggen
G. A. Fiorelli

(48) P. Harbut

Arizona Public Service Company RECEIVED
P.O. BOX 21666 • PHOENIX, ARIZONA 85036

March 14, 1983 1:00
ANPP-23238-RQT/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

REGION V

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-9
A 50.55(e) Potentially Reportable Deficiency Relating to
Unit 1 LPSI Pump "A" Suction Strainer Contains Foreign
Material, And Pump "B" Strainer Is Torn
File: 83-019-026
D.4.33.2

Reference: (A) Telephone Conversation between T. Young and
G. Duckworth on February 16, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the
referenced telephone conversation. At that time, it was estimated that a
determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by May 12, 1983, at which time a complete report will be
submitted.

Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/RQT:db
Enclosure
cc: See Attached Page 2

8304040388

#48

U. S. Nuclear Regulatory Commission
Attention: Mr. D. M. Sternberg, Chief
Page 2

March 14, 1983
ANPP-23238-RQT/BSK

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
J. A. Roedel
D. B. Fasnacht
G. C. Andognini
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G. A. Fiorelli

INTERIM REPORT - DER 83-9
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. POTENTIAL PROBLEM

Startup Quality Assurance Stop Work Notice S-83-005 documents two (2) findings regarding the cleanliness level of the Low Pressure Safety Injection (LPSI) system as follows:

1. A substantial amount of foreign material (purge dam material, etc.) was discovered in the suction strainer for Low Pressure Safety Injection Pump (LPSI) 1MSIAP01 after operation.
2. A torn suction strainer for Low Pressure Safety Injection Pump (LPSI) 1MSIBP01 was discovered. The torn strainer indicates that a foreign object may have entered the LPSI pump.

The reliability of the LPSI system is consequently indeterminate, and continued operation without corrective action could result in possible damage to the LPSI pumps.

II. APPROACH TO AND STATUS OF PROPOSED RESOLUTION

Damage to the suction strainer of LPSI Pump 1MSIBP01 has been evaluated and is currently being replaced with a better design. Bechtel Engineering is performing an investigation to determine the cause of entry of foreign material into the LPSI system.

III. PROJECTED COMPLETION OF CORRECTIVE ACTION
AND SUBMITTAL OF THE FINAL REPORT

Evaluation of this condition and submittal of the Final Report is forecast to be completed by May 12, 1983.

ARIZONA

STA. 3003



PUBLIC SERVICE COMPANY

P.O. BOX 21666 - PHOENIX, ARIZONA 85036

September 29, 1982

ANPP-21921-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Final Report - **DER 82-35**
A 50.55(e) Report Relating to Unit 2 LPSI and
Containment Spray Pump Casing Hold-Down Bolts
Under-Torqued
File: 82-019-026; D.4.33.2

Reference: (A) Telephone Conversation between J. Eckhardt and
G. Duckworth on July 7, 1982
(B) ANPP-21557, dated August 5, 1982 (Interim Report)

Dear Sir:

Attached is our final written report of the deficiency referenced
above, which has been determined to be Not Reportable under the
requirements of 10CFR50.55(e).

Very truly yours,

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/GHD:db

Attachment

cc: See Attached Page 2

8210130495

#50

U. S. Nuclear Regulatory Commission
Attention: Mr. D. M. Sternberg, Chief
Page 2

September 29, 1982
ANPP-21921-GHD/BSK

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
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G. C. Andognini
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G. A. Fiorelli

Lynne Bernabei, Esq.
Harmon & Weiss
1725 "I" Street, NW
Suite 506
Washington, D. C. 20006

R. L. Greenfield
Assistant Attorney General
Bataan Memorial Building
Santa Fe, New Mexico 87503

FINAL REPORT - DER 82-35

DEFICIENCY EVALUATION 50.55(e)

ARIZONA PUBLIC SERVICE COMPANY (APS)

PVNGS UNIT 2

I. DESCRIPTION OF DEFICIENCY

Ingersoll-Rand's instruction manual specifies a 4,000 ft.-lbs. torque value for the Low Pressure Safety Injection (LPSI) and Containment Spray Pumps hold down bolts. During the conduct of rework of the LPSI "B" Pump (087641) and Containment Spray "B" Pump (S/N 087645) per requirements of NCR NA-080, a hydraulic torque wrench (direct read-out) was utilized to remove the nuts, revealing that the break-away torque was 2,500 ft.-lbs. This DER was initiated to evaluate the condition, since the original application of tightening torque was done utilizing a field fabricated wrench, and may not have provided the necessary accuracy.

It is Engineering's position that the 2,500 ft.lb. torque value was not an accurate measure of the original tightening torque. This position is based on the following findings:

1. Per references (1) and (2) the amount of torque required to start disassembly of an axially loaded fastener is called the break loose torque, and is less than the torque required to tighten. In reference (2), Power and Engineered Products Company (PEPCO) performed tests to determine the torque required for disassembly of coated (lubricant) and uncoated bolts. For the lubricated bolts the break loose torque varied between fifty-three (53) and Sixty-five (65) percent of the original tightening torque. Similarly, the uncoated samples varied between seventy-three (73) and seventy-eight (78) percent. The Ingersoll Rand Installation Manual (Reference 3) calls for the application of a lubricant to the bolts to prevent galling.
2. Additionally, there is a drop in bolt tension versus time. This condition is known as relaxation resulting from the elastic recovery which takes place when the torque wrench is removed. (Torsional stresses go to zero). Secondly, creep and yielding in the bolt due to the high stress level at the root of the threads results in further relaxation over time. Therefore, the break loose torque is not an accurate method for measuring the original hold down torque. Dependent on many factors, the break loose may vary between fifty (50) and seventy-five (75) percent of the assembly torque. A 2,500 ft.-lb. or 62.5 percent disassembly torque for the LPSI pump casing bolts falls within this range.

II. ANALYSIS OF SAFETY IMPLICATIONS

APS and Bechtel Engineering have evaluated the torque value recorded during the rework of the LPSI "B" pump, and conclude that this condition is not reportable under the requirements of 10CFR50.55(e). The observed value of break loose torque is indicative of an original hold down torque in compliance with the specification requirement.

III. CORRECTIVE ACTION

A copy of this DER will be routed to the responsible field engineers to assure understanding of the methodology used in verifying fastener torque values. In the future, field personnel will verify a torque value by recording the additional tightening value rather than the break loose value.

REFERENCES

1. "Helpful Hints for Fastener Design and Application", Russell, Burdsall, Ward Corporation.
2. "Report from G. P. Miller to J. Stravinkas", Power and Engineered Products Company, Incorporated, June 21, 1982.
3. Instruction Manual for LPSI Pumps, Ingersoll-Rand Bechtel Log No. N001-9.02-1.

ARIZONA

STA. 3003



PUBLIC SERVICE COMPANY

P.O. BOX 21666 - PHOENIX, ARIZONA 85036

August 5, 1982

ANPP-21557-GHD/BSK

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U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. T. W. Bishop, Chief
Reactor Construction Projects Branch

Subject: Interim Report - DER 82-35
A 50.55(e) Potentially Reportable Deficiency Relating to
Unit 2 LPSI and Containment Spray Pump Casing
Hold-Down Bolts Under-Torqued.
File: 82-019-026
D.4.33.2

Reference: Telephone Conversation between J. Eckhardt and G. Duckworth
on July 7, 1982

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the
referenced telephone conversation. At that time, it was estimated that
a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by September 30, 1982, at which time a complete report will be
submitted.

Very truly yours,

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects
ANPP Project Director

EEVBJr/GHD:db

Attachment

cc: See Attached Page 2

8208160281

#51

U. S. Nuclear Regulatory Commission
Attention: Mr. T. W. Bishop, Chief

ANPP-21557-GHD/BSK
August 5, 1982

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

J. A. Roedel
D. B. Fasnacht
G. C. Andognini
A. C. Rogers
B. S. Kaplan
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J. Vorees
J. A. Brand
A. C. Gehr
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G. A. Fiorelli

Lynne Bernabei, Esq.
Harmon & Weiss
1725 "I" Street, NW
Suite 506
Washington, D. C. 20006

R. L. Greenfield
Assistant Attorney General
Bataan Memorial Building
Santa Fe, New Mexico 87503

INTERIM REPORT - DER 82-35

POTENTIAL REPORTABLE DEFICIENCY

ARIZONA PUBLIC SERVICE COMPANY (APS)

I. POTENTIAL PROBLEM

Ingersoll-Rand Instruction Manual specified 4,000 ft. lbs. torque for the Low Pressure Safety Injection (LPSI) and Containment Spray Pumps hold down bolts. During rework of the LPSI "B" Pump (087641) and Containment Spray "B" Pump (S/N 087645) per requirements of NCR NA-080, a hydraulic torque wrench (direct read-out) was utilized to remove the nuts, revealing that the break-away torque was 2,500 ft. lbs.

Subsequent to this discovery, research revealed that the original torque was applied utilizing a field fabricated wrench, torque wrench and multiplier. During the calibration process, the torque wrench was the only item calibrated. When torque was applied, the multiplier ratio was used to determine the torque applied. This application may not have provided the desired accuracy.

II. APPROACH TO AND STATUS OF PROPOSED RESOLUTION

Engineering is performing an analysis of this condition to determine:

- (1) If the nut removal torque is an accurate measure of the original tightening torque.
- (2) If the 2,500 ft. lbs. torque is sufficient to meet the operational requirements of the pump and pump motor.

III. PROJECTED COMPLETION OF CORRECTIVE ACTION
AND SUBMITTAL OF THE FINAL REPORT

Evaluation of this condition and submittal of the Final Report is forecast to be completed by September 30, 1982.

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ARIZONA

STA. 3003



PUBLIC SERVICE COMPANY

P.O. BOX 21666 - PHOENIX, ARIZONA 85036

May 26, 1982
ANPP-21044-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. T. W. Bishop, Chief
Reactor Construction Projects Branch

Subject: Final Report - DER 81-52
A 50.55(e) Report Relating to Unit 1 Low Pressure
Safety Injection Pumps and Containment Spray Pumps
Have Potentially Loose Anti-Rotation Pins
File: 82-019-026
D.4.33.2

Reference: (A) Telephone Conversation between J. Eckhardt and
G. Duckworth on December 30, 1981
(B) ANPP-20014, dated January 28, 1982 (Interim
Report)
(C) ANPP-20486, dated March 19, 1982 (Time Extension)
(D) ANPP-20818, dated April 27, 1982 (Time Extension)

Dear Sir:

Attached, is our final written report of the deficiency referenced
above, which has been determined to be Not Reportable under the
requirements of 10CFR50.55(e).

Very truly yours,

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects
ANPP Project Director

EEVBJr/GHD:skc

Attachment

cc: See Attached Page 2

8206150139

#52

82-144

U. S. Nuclear Regulatory Commission
Attention: Mr. T. W. Bishop, Chief
ANPP-21044-GHD/BSK
May 26, 1982
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

J. A. Roedel
D. B. Fasnacht
G. C. Andogaini
A. C. Rogers
B. S. Kaplan
W. E. Ide
J. Vorees
J. A. Brand
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G. A. Fiorelli

Lynne Bernabei, Esq.
Harmon & Weiss
1725 "I" Street, NW
Suite 506
Washington, D. C. 20006

R. L. Greenfield
Assistant Attorney General
Bataan Memorial Building
Santa Fe, New Mexico 87503

FINAL REPORT - DER 81-52
DEFICIENCY EVALUATION 50.55(e)
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNITS 1

I. Description of Deficiency

Combustion Engineering identified a low pressure safety injection pump failure at the Arkansas One plant. Subsequent Combustion Engineering correspondence requested Bechtel to check and reinstall any loose Anti-Rotation Pins in the low pressure safety injection and containment spray pumps manufactured by Ingersoll-Rand.

II. Analysis of Safety Implications

This condition is evaluated as Not Reportable based on the attached Combustion Engineering Letter V-CE-15903, dated February 10, 1982. Should this condition have remained undetected and, if uncorrected, it would not represent a safety significant condition.

III. Corrective Action

Nonconformance Report NA638 (Low Pressure Safety Injection Pump) will be dispositioned to secure the loose pins by tack welding in compliance with Combustion Engineering Letters V-CE-6180, dated March 15, 1980, and V-CE-6354, dated October 8, 1981.

Nonconformance Report NA639 (Containment Spray Pump) will be dispositioned as Use-As-Is, since the pins were found to be tight by Bechtel Field personnel.

The above pumps for Unit 1 were modified under Ingersoll-Rand supervision. The pump sets for Units 2 and 3 were modified by Ingersoll-Rand prior to delivery to the jobsite (see attached Letter I-R to W. G. Bingham, dated April 16, 1982).

C-E Power Systems
Combustion Engineering, Inc.
1000 Prospect Hill Road
Windsor, Connecticut 06095

Tel. 203/688-1911
Telex: 99297

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NS DOL. CE

CE POWER SYSTEMS

February 10, 1982
V-CE-15903

Mr. W. G. Bingham
Bechtel Power Corporation
12400 East Imperial Highway
Norwalk, California 90650

DER 81-52

Subject: ANPP Bechtel Job 10407
Anti-rotation Pins LPSI CSPumps

References: (A) B/CE-E-37869, dated December 31, 1981
(B) Bechtel Deficiency Evaluation Report No. 81-52
dated 12/17/81.
(C) CE letter V-CE-8313, dated 5/25/79

Dear Mr. Bingham:

Reference (A) forwarded Reference (B) to CE and asked for CE input as to the reportability of the reported deficiency. CE does not consider the loose fitting "anti-rotation pin" situation to be a reportable deficiency item for the following reasons:

1. The pins are all to be checked for tightness at the site prior to pump operation. Any pins discovered to be loose are to be reinstalled in accordance with the procedure in Reference (C) above.
2. If a pin were to come loose it would remain captive in the region between the impeller and diffuser. This event occurred at Arkansas. Noise of the pin rattling between impeller and diffuser was detected and the pump shut down and disassembled. The pin which was originally about 1 1/2" long by 1/2" diameter had been reduced to the size of a pencil eraser.
3. The pin is not designed to carry any loads, but is merely a device to properly locate the diffuser during pump assembly. The diffuser is secured in position, between the stuffing box extension and the suction ring, when the main casing bolting is torqued, thereby preventing its rotation.

If you have any questions, please contact us.

Very truly yours,

C. Ferguson
Project Manager

CF/CDB/rre

cc: Messrs:

E. E. Van Brunt, Jr.
F. W. Hartley
W. H. Wilson

R. H. Holm
W. L. MacDonald
G. A. Butterworth

S. K. Mager
G. C. Andognini
D. B. Amerine

*To Assign 9.1	
BSK	
Resp.	JMA
Mgr./	ACG
Supv	GCA
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	*JAR
	ACR
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	NEH
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82-001-419.1 enc.	



April 16, 1982

Bechtel Power Corporation
P.O. Box 60860 - Terminal Annex
Los Angeles, CA 90060

Attention: Mr. William Bingham

Subject: ANPP-Palo Verde Nuclear Generating Station
Units 1, 2 and 3
Model 8x20 WDF Low Pressure Safety Injection Pumps
I-R Orders 001-36423/26/29
and
Model 8x23 WDF Containment Spray Pumps
I-R Orders 001-36424/27/30

Gentlemen:

We respond to your recent request for clarification of the change in the anti-rotation pin to prevent the diffusor from rotating in the pump casing.

On Unit #1 both sets of pumps were modified by captive welding the pin under Ingersoll-Rand supervision.

On Units #2 and #3 the pump sets were shipped later and we were able to incorporate a design change during manufacture so that the pin is permanently secured and no field modification of these pumps is required.

We hope this clarifies the situation.

Very truly yours,

INGERSOLL-RAND COMPANY

Ted R. Kernkamp

mm

Los Angeles Office

Ingersoll-Rand Company
5533 East Olympic Boulevard
Los Angeles, CA 90022-0048
(213) 723-0131

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ARIZONA



PUBLIC SERVICE COMPANY

P. O. BOX 21666 • PHOENIX, ARIZONA 85036

March 19, 1982
ANPP-20486-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. B. H. Faulkenberry, Chief
Reactor Construction and
Engineering Support Branch

Subject: Time Extension for Report - DER 81-52
A 50.55(e) Potentially Reportable Deficiency Relating
to Unit 1 Low Pressure Safety Injection Pumps and
Containment Spray Pumps Have Potentially Loose Anti-
Rotation Pins
File: 82-019-026
D.4.33.2

Reference: (A) Telephone Conversation between J. Eckhardt and
G. Duckworth on December 30, 1981
(B) ANPP-20014, dated January 28, 1982 (Interim Report)

Dear Sir:

The NRC was notified of a potentially reportable deficiency in
Reference (A) and an Interim Report was transmitted by Reference (B).
At that time, it was estimated that a Final Report would be available
by March 23, 1982.

Due to the extensive investigation and evaluation required, it is now
expected that this information will be finalized by April 30, 1982,
when a complete report will be submitted.

There is no new information or change to the Interim Report at this time.



Very truly yours,

E. E. Van Brunt

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects
ANPP Project Director

EEVBjr/GHD:skc

cc: See Attached Page 2

~~4204050320~~

#53 82-68

U. S. Nuclear Regulatory Commission
Attention: Mr. B. H. Faulkenberry, Chief
ANPP-20486-GHD/BSK
March 19, 1982
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

J. A. Roedel
D. B. Fasnacht
G. C. Andognini
F. W. Hartley
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D. R. Hawkinson
L. E. Vorderbrueggen

R. L. Greenfield
Assistant Attorney General
Bataan Memorial Building
Santa Fe, New Mexico 87503

Ms. Patricia Lee Hourihan
6413 South 26th Street
Phoenix, Arizona 85040

ARIZONA



PUBLIC SERVICE COMPANY

P. O. BOX 21666 • PHOENIX, ARIZONA 85036

April 27, 1982
ANPP-20818-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. T. W. Bishop, Chief
Reactor Construction and
Engineering Support Branch

Subject: Time Extension for Report - DER 81-52
A 50.55(e) Potentially Reportable Deficiency Relating
to Unit 1 Low Pressure Safety Injection Pumps and
Containment Spray Pumps Have Potentially Loose Anti-
Rotation Pins
File: 82-019-026
D.4.33.2

Reference: (A) Telephone Conversation between J. Eckhardt and
G. Duckworth on December 30, 1981
(B) ANPP-20014, dated January 28, 1982 (Interim Report)
(C) ANPP-20486, dated March 19, 1982 (Extension)

Dear Sir:

The NRC was notified of a potentially reportable deficiency in Reference (A). An Interim Report was transmitted by Reference (B), and a time extension was requested by Reference (C). At that time, it was estimated that a Final Report would be available by April 30, 1982.

Due to the extensive investigation and evaluation required, it is now expected that this information will be finalized by May 28, 1982, when a complete report will be submitted.

There is no new information or change to the Interim Report at this time.

Very truly yours,

E. E. Van Brunt, Jr.

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects
ANPP Project Director

EEVBJr/GHD:skc

cc: See Attached Page 2

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MAY -3 PM 1:23

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U. S. Nuclear Regulatory Commission
Attention: Mr. T. W. Bishop, Chief
ANPP-20818-GHD/BSK
April 27, 1982
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

J. A. Roedel
D. B. Fasnacht
G. C. Andognini
A. C. Rogers
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ARIZONA



PUBLIC SERVICE COMPANY

P. O. BOX 21666 • PHOENIX, ARIZONA 85036

January 28, 1982
ANPP-20014-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. B. H. Faulkenberry, Chief
Reactor Construction and
Engineering Support Branch

Subject: Interim Report - DER 81-52
A 50.55(e) Potentially Reportable Deficiency Relating
to Unit 1 Low Pressure Safety Injection Pumps and
Containment Spray Pumps Have Potentially Loose Anti-
Rotation Pins
File: 82-019-026
D.4.33.2

Reference: Telephone Conversation between J. Eckhardt and G. Duckworth
on December 30, 1981

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the referenced telephone conversation. At that time, it was estimated that a determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim Report is attached. It is now expected that this information will be finalized by March 23, 1982, at which time a complete report will be submitted.

Very truly yours,

E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects
ANPP Project Director

EEVBJr/GHD:skc

Attachement

cc: See Attached Page 2

RECEIVED

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82-17

U. S. Nuclear Regulatory Commission
Attention: Mr. B. H. Faulkenberry, Chief
ANPP-20014-GHD/BSK
January 28, 1982
Page 2

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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INTERIM REPORT - DER 81-52
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. Potential Problem

Combustion Engineering identified a Low Pressure Safety Injection Pump failure at the Arkansas One plant. Subsequent Combustion Engineering correspondence requested Bechtel to check and reinstall any loose Anti-Rotation Pins in the Low Pressure Safety Injection and Containment Spray Pumps manufactured by Ingersoll-Rand.

II. Approach To and Status Of Proposed Resolution

Bechtel has requested assistance from Combustion Engineering in evaluating and dispositioning this item.

III. Projected Completion of Corrective Action and Submittal of the Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by March 23, 1982.