



Duquesne Light

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May 22, 1984

United States Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Mr. George W. Knighton, Chief
Licensing Branch 3
Office of Nuclear Reactor Regulation

SUBJECT: Beaver Valley Power Station - Unit No. 2
Docket No. 50-412
S.E.R. Outstanding Issues Status

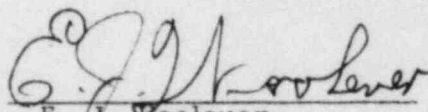
Gentlemen:

Attachment 1 is a current list, as of May 15, 1984, which provides our understanding of outstanding issues identified in Table 1.2 of the draft Safety Evaluation Report. Attachment 2 is a current list, as of May 15, 1984, which provides our understanding of confirmatory issues identified in Table 1.3 of the draft Safety Evaluation Report. Items identified as "complete" are those for which responses have been provided and no confirmation of status has yet been received from the staff. We consider these items satisfactorily closed unless notified otherwise. In order to permit timely resolution of items identified as "complete" which may not be resolved to the staff's satisfaction, please provide a specific description of the issue which remains to be resolved.

Items identified as "closed" or "confirmatory" are based upon formal or informal communications or agreements with the staff. Please inform us of any differing opinion of status described on the attached list.

Attachment 3 is a current list, as of May 15, 1984, which identifies draft SER sections not provided in the March 1, 1984, issue of the draft SER.

DUQUESNE LIGHT COMPANY

By 
E. J. Woolever
Vice President

GLB/wjs
Attachment

cc: Mr. H. R. Denton, Director NRR (w/a)
Mr. D. Eisenhat, Director Division of Licensing (w/a)
Mr. T. Novak, Assistant Director Division of Licensing (w/a)
Mr. G. Walton, NRC Resident Inspector (w/a)
Mr. M. Licitra, Project Manager (w/a)

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ATTACHMENT 1

OUTSTANDING ISSUES

| OUTSTANDING ISSUE | SUBJECT | PROPOSED RESPONSE DATE | STATUS | LETTER NO. AND DATE |
|----------------------|--|------------------------------|----------|---------------------|
| 1 | Potential for flooding from local intense precipitation | | 1 | |
| 2 | Flooding from Peggs Run | | 1 | |
| 3 | Adequacy of the proposed tech spec for drought conditions | 4/30/84 | Complete | 2NRC-4-049, 4/30/84 |
| 4 | Adequacy of intake structure silt monitoring program | 4/30/84 | Complete | 2NRC-4-049, 4/30/84 |
| 5 | Boundaries of the Appalachian Plateau Tectonic Province | 4/30/84 | Complete | 2NRC-4-048, 4/27/84 |
| 6 | Maximum earthquake | 4/30/84 | 2 | 2NRC-4-048, 4/27/84 |
| 7 | Ground motion taking soil properties into consideration | 4/30/84 | 2 | 2NRC-4-048, 4/27/84 |
| 8 | Actual earthquake time histories | 4/30/84 | 2 | 2NRC-4-048, 4/27/84 |
| 9 | Shallow earthquakes | 4/30/84 | Complete | 2NRC-4-048, 4/27/84 |
| 10 | Vertical seismic design accelerations | 4/30/84 | 2 | 2NRC-4-048, 4/27/84 |
| 11 | Tornado pressure drop rate calculations | 4/30/84 | Complete | 2NRC-4-047, 4/27/84 |
| 12 | Concrete missile barrier design | | Complete | 2NRC-4-018, 2/27/84 |
| 13 | Site-specific response spectra | 4/30/84 | 2 | 2NRC-4-047, 4/27/84 |
| 14 | Justification of peak spreading of floor response spectra | | Complete | 2NRC-4-018, 2/27/84 |
| 15 | 3-component seismic input vs. 2-component input | 6/15/84 | | |
| 16 | Soil-structure interaction analysis of containment and intake structure | 6/15/84 | | |
| 17 | Significance of the coupling in structural response between mutually orthogonal axes of structures | 6/15/84 | | |
| 18 | Deviations of containment design from ASME Code Section III Division 2 | 6/15/84 | | |
| 19 | Ultimate capacity analysis of containment | 6/15/84 | | |
| 20 | Deviations from requirements of ACI 349 code as augmented by Reg. Guide 1.142 | 4/27/84 | Complete | 2NRC-4-047, 4/27/84 |
| 21 | Structural audit action items | 4/27/84 | Complete | 2NRC-4-047, 4/27/84 |
| 22 | Affects of cracked panel on dynamic modeling and design | | Complete | 2NRC-4-018, 2/27/84 |
| 23 | Adequacy of re-assessed safety factors against sliding and overturning of containment, aux. building, and intake structure | 4/30/84 | Complete | 2NRC-4-047, 4/27/84 |
| 24 | Through-wall leakage cracks in moderate energy lines in containment | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 25 | Selection of postulated break locations | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 26 | Jet impingement effects | 6/29/84 | 2 | 2NRC-4-4052, 5/8/84 |

| OUTSTANDING ISSUE | SUBJECT | PROPOSED RESPONSE DATE | STATUS | LETTER NO. AND DATE |
|----------------------|--|------------------------------|--------------|---------------------|
| 27 | Break exclusion zones | 5/15/84 | Confirmatory | 2NRC-4-4052, 5/8/84 |
| 28 | Pipe-to-pipe impact | 5/15/84 | Complete | 2NRC-4-4052, 5/8/84 |
| 29 | Limited break areas | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 30 | Saturated or subcooled water blowdown | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 31 | Design of pipe rupture restraints | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 32 | List of transients | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 33 | Location of systems to be monitored during pre-op testing | 6/01/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 34 | Criteria used for determining acceptability of vibration levels | 5/04/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 35 | Vibration monitoring program | 5/04/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 36 | Combining 3 components of earthquake motion | 5/04/84 | Complete | 2NRC-4-4052, 5/8/84 |
| 37 | Modes included in seismic analyses | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 38 | Demonstrating seismic qualification | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 39 | Loading combinations, system operating transients, and stress limits | 5/15/84 | 2 | 2NRC-4-4052, 5/8/84 |
| 40 | HVAC system design | 5/15/84 | 2 | 2NRC-4-4052, 5/8/84 |
| 41 | Design of safety and relief valves | 5/15/84 | Closed | 2NRC-4-4052, 5/8/84 |
| 42 | Design and construction of ASME Class 1, 2, and 3 component support | 5/15/84 | Complete | 2NRC-4-4052, 5/8/84 |
| 43 | Preservice and inservice testing of pumps and valves | a-6/84 b-12/84 c-6/85 | 2 | 2NRC-4-4052, 5/8/84 |
| 44 | Effects of rod bow on DNB | 6/29/84 | | |
| 45 | Flow measurement capability and procedure | 6/29/84 | | |
| 46 | Loose parts detection program report | 6/29/84 | | |
| 47 | Documentation of complete ICC system | 6/29/84 | | |
| 48 | Preservice inspections program | a-6/84 | | |
| 49 | Compliance with Appendix G 10CFR Part 50 | 6/29/84 | Complete | 2NRC-4-049, 4/30/84 |
| 50 | Compliance with Appendix H 10CFR Part 50 | 4/30/84 | Complete | 2NRC-4-049, 4/30/84 |
| 51 | Pressure temperature limits | 4/30/84 | Complete | 2NRC-4-049, 4/30/84 |
| 52 | | | 3 | |
| 53 | Control room habitability | 6/01/84 | | |
| 54 | Inservice inspection of Class 2 and 3 components | a-6/84 c-6/85 | | |
| 55 | Design modification for automatic reactor trip | | Complete | 2NRC-4-033, 3/30/84 |
| 56 | Anticipatory reactor trip on turbine trip | 5/25/84 | | |

| OUTSTANDING ISSUE | SUBJECT | PROPOSED RESPONSE DATE | STATUS | LETTER NO. AND DATE |
|----------------------|---|------------------------------|----------|---------------------|
| 57 | P-4 interlock | | Complete | 2NRC-4-038, 4/10/84 |
| 58 | Undetectable failure in online testing circuitry for engineered safeguards relays | 6/29/84 | | |
| 59 | Service water system isolation on low header pressure | 5/31/84 | | |
| 60 | Normal letdown line relief valve | 5/25/84 | | |
| 61 | Switchover from injection to recirculation | 5/11/84 | Complete | 2NRC-4-056, 5/14/84 |
| 62 | Main feedwater isolation | 5/25/84 | | |
| 63 | Control room isolation | 6/29/84 | | |
| 64 | Steam generator level control and protection | | Complete | 2NRC-4-032, 3/28/84 |
| 65 | IE Bulletin 80-06 concerns | 6/29/84 | | |
| 66 | Independence between manual and automatic action | 6/29/84 | | |
| 67 | Power lockout for motor-operated valves | | Complete | 2NRC-4-032, 3/28/84 |
| 68 | Remote shutdown capability | 5/25/84 | | |
| 69 | Emergency response capability | | 5 | |
| 70 | Direct indication of relief and safety valve positions | 5/18/84 | | |
| 71 | Bypass and inoperable status panel | 5/25/84 | | |
| 72 | IE Bulletin 79-27 | 5/25/84 | | |
| 73 | Reactor coolant system loop isolation interlocks | 6/29/84 | | |
| 74 | Primary component cooling water isolation from RCP thermal barriers | 6/29/84 | | |
| 75 | PID controller modification | 6/29/84 | | |
| 76 | High energy line breaks and consequential control system failures | 4/27/84 | Complete | 2NRC-4-040, 4/30/84 |
| 77 | Control system failure caused by malfunctions of common power source or instrument line | 5/25/84 | | |
| 78 | Procedure to estimate extent of core damage | | Complete | 2NRC-4-042, 4/18/84 |
| 79 | Backup post accident sampling through grab samples for inline analyses | | Complete | 2NRC-4-042, 4/18/84 |
| 80 | Measuring radionuclide concentrations | | Complete | 2NRC-4-042, 4/18/84 |
| 81 | Performance of PASS instrumentation and analytical procedures | | Complete | 2NRC-4-042, 4/18/84 |
| 82 | Fire hazards analysis | 5/15/84 | | |
| 83 | Fire brigade | 5/15/84 | | |
| 84 | Penetration seals | 5/15/84 | | |
| 85 | Safe shutdown | | 5 | |
| 86 | Alternate shutdown | | 5 | |

| OUTSTANDING ISSUE | SUBJECT | PROPOSED RESPONSE DATE | STATUS | LETTER NO. AND DATE |
|----------------------|---|------------------------------|----------|---------------------|
| 87 | Hydrogen piping | 5/15/84 | | |
| 88 | Cable tray suppression | 5/15/84 | | |
| 89 | Power supplies for control room ventilation | 5/15/84 | | |
| 90 | Fire detection | 5/ '84 | | |
| 91 | Valve supervision | 5/15/84 | | |
| 92 | Reactor coolant pumps and separation of safety-related components in containment | 5/15/84 | | |
| 93 | Control room complex | 5/15/84 | | |
| 94 | Cable spreading room | 5/1 /84 | | |
| 95 | Exemption to 10CFR 70.24(-) and description of alternative to required criticality monitors | 6/01/84 | | |
| 96 | Types, numbers, specifications for portable and laboratory HP instruments | 5/15/84 | | |
| 97 | Training/retraining for health physics professionals and verification that contractor training meets 10CFR 19.12 requirements | | 6 | |
| 98 | Procedures generation package/TMI I.C.1, short-term accident analysis and procedures revision | | 7 | |
| 99 | Physical security | 5/16/84 | Complete | 2NRC-4-051, 5/04/84 |
| 100 | Initial test program | 5/30/84 | | |
| 101 | Radiological consequences of control rod ejection DBA | | 5 | |
| 102 | Radiological consequences of a small line break DBA | 5/15/84 | | |
| 103 | Radiological consequences of a steam generator tube rupture DBA and review of operator actions and system performance | 6/29/84 | | |
| 104 | Radiological consequences of a steamline break outside secondary containment | | 5 | |
| 105 | Radiological consequences of a loss of coolant DBA | | 5 | |
| 106 | Radiological consequences of a fuel handling DBA | | 5 | |
| 107 | | | 3 | |
| 108 | | | 3 | |
| 109 | | | 3 | |
| 110 | | | 3 | |
| 111 | | | 3 | |
| 112 | Technical specifications | | 8 | |
| 113 | Detailed control room design review | 6/01/85 | | |
| 114 | Safety parameter display system | 8/01/84 | | |

| <u>OUTSTANDING ISSUE</u> | <u>SUBJECT</u> | <u>PROPOSED RESPONSE DATE</u> | <u>STATUS</u> | <u>LETTER NO. AND DATE</u> |
|------------------------------|---|---------------------------------------|---------------|----------------------------|
| 115 | Snowpack load on roofs | 6/29/84 | | |
| 116 | Extreme temperatures for site area | 6/29/84 | | |
| 117 | Affects of relocation of met. tower | 6/29/84 | | |
| 118 | Long term diffusion estimates | 6/29/84 | | |
| 119 | Deviations to R.G. 1.52 | 6/29/84 | | |
| 120 | Containment vacuum system exhaust iodine removal capability | 6/29/84 | | |
| 121 | Evaluation of RAST failure | 6/29/84 | 5 | |

¹Currently evaluating letter from T. M. Novak to E. J. Woolever dated April 11, 1984

²Supplemental submittal to be provided.

³Not provided in draft SER

⁴Further resolution being discussed informally

⁵Staff action only

⁶To be discussed at RAB meeting 5/01/84

⁷No response required

⁸DLC is currently arranging a management meeting with NRC to discuss this item

^aPSI PROGRAM

^bPSI PROCEDURES FOR VALVES & PUMPS

^cISI PROGRAM

ATTACHMENT 2

CONFIRMATORY ISSUES

| OUTSTANDING ISSUE | SUBJECT | STATUS | LETTER NO. AND DATE |
|----------------------|--|----------|---------------------|
| 1 | Operating basis earthquake | Complete | 2NRC-4-047, 4/27/84 |
| 2 | Stress and strain levels of key structural elements | | |
| 3 | Floor response spectra accounting for 3-component earthquake input | | |
| 4* | | | |
| 5 | Examination of steam generators | | |
| 6* | | | |
| 7 | Main FW isolation | | |
| 8 | CR isolation on high radiation signal | | |
| 9 | Automatic opening of service water system valves | | |
| 10 | Accident monitoring instrumentation positions | | |
| 11 | Cold leg accumulator MOV position indication | | |
| 12 | Table 1.9-2 by-product, source, and special nuclear material descriptions nine months prior to fuel loading Q471.14B | | |
| 13 | Verification that cobalt use in primary system has been minimized Q471.10 | | |
| 14 | Containment high range radiation monitors six months prior to fuel loading Q471.14a | | |
| 15 | Verify that RMS airborne radioactivity monitors | | |
| 16 | Quality assurance program requirements | | |

*Not provided in Draft SER

ATTACHMENT 3

DRAFT SER SECTIONS AND DATES PROVIDED

| <u>Section</u> | <u>Date</u> | <u>Section</u> | <u>Date</u> | <u>Section</u> | <u>Date</u> | <u>Section</u> | <u>Date</u> | <u>Section</u> | <u>Date</u> |
|----------------|-------------|----------------|-------------|----------------|-------------|----------------|-------------|----------------|-------------|
| 2.1.1 | | 5.2.3 | | 8.4.4 | | 10.4.4 | | 15.3.3 | |
| 2.1.2 | | 5.2.5 | | 8.4.5 | | 10.4.5 | | 15.3.4 | |
| 2.1.3 | | 5.4.1 | | 8.4.6 | | 10.4.7 | | 15.4.5 | |
| 2.1.4 | | 5.4.7 | | 8.4.7 | | 10.4.9 | | 15.4.6 | |
| 2.2.1 | | 5.4.11 | | 8.4.8 | | 11 | 05/04/84 | 15.5.1 | |
| 2.2.2 | | 6.2.1 | | 9.1.1 | | 11.1 | 05/04/84 | 15.5.2 | |
| 2.2.3 | | 6.2.2 | | 9.1.4 | | 11.1.2 | | 15.7.1 | |
| 2.3.1 | 05/04/84 | 6.2.3 | | 9.1.5 | | 11.2.1 | 05/04/84 | 15.7.2 | |
| 2.3.2 | 05/04/84 | 6.2.4 | | 9.2.1 | | 11.2.2 | 05/04/84 | 15.7.3 | 05/04/84 |
| 2.3.3 | 05/04/84 | 6.2.5 | | 9.2.2 | | 11.3.1 | 05/04/84 | 15.9.1 | |
| 2.3.4 | 05/04/84 | 6.2.6 | | 9.2.3 | | 11.3.2 | 05/04/84 | 15.9.2 | |
| 2.3.5 | 05/04/84 | 6.3.1 | | 9.2.4 | | 11.3.3 | 05/04/84 | 15.9.3 | |
| 2.4.13 | | 6.3.2 | | 9.2.5 | | 11.4.1 | | 15.9.4 | |
| 2.5.4 | | 6.3.3 | | 9.2.6 | | 11.4.2 | | 15.9.5 | |
| 2.5.5 | | 6.3.4 | | 9.3.1 | | 11.5.1 | 05/04/84 | 15.9.6 | |
| 2.5.6 | | 6.3.5 | | 9.3.3 | | 11.5.2 | 05/04/84 | 15.9.7 | |
| 3.2.1 | | 6.3.6 | | 9.4.1 | | 13.1.1 | | 15.9.8 | |
| 3.2.2 | | 6.5.1 | 05/04/84 | 9.4.2 | | 13.1.2 | | 15.9.9 | |
| 3.4.1 | | 6.5.3 | | 9.4.3 | | 13.3 | | 15.9.10 | |
| 3.5.1 | | 6.5.4 | | 9.4.4 | | 13.4 | | 15.9.11 | |
| 3.5.2 | | 7.2.3 | | 9.4.5 | | 13.5.1 | | 15.9.12 | |
| 3.6.1 | | 7.3.4 | | 9.5.2 | | 15.1.2 | | 15.9.13 | |
| 3.10.1 | | 7.4.3 | | 9.5.3 | | 15.1.3 | | 15.9.14 | |
| 3.10.2 | | 7.5.3 | | 9.5.4 | | 15.1.4 | | 17.1 | |
| 3.11.1 | | 7.6.3 | | 9.5.5 | | 15.1.5 | | 17.2 | |
| 3.11.2 | | 7.7.3 | | 9.5.6 | | 15.2.1 | | 17.3 | |
| 3.11.3 | | 8.2.1 | | 9.5.7 | | 15.2.2 | | 17.4 | |
| 4.2.1 | | 8.2.2 | | 9.5.8 | | 15.2.3 | | 19 | |
| 4.2.2 | | 8.2.3 | | 10.2.3 | | 15.2.4 | | 20 | |
| 4.2.3 | | 8.2.4 | | 10.3.1 | | 15.2.5 | | 21 | |
| 4.2.4 | | 8.3.1 | | 10.3.2 | | 15.2.6 | | 22.1 | |
| 4.2.5 | | 8.3.2 | | 10.3.6 | | 15.2.7 | | 22.2 | |
| 4.5.1 | | 8.4.1 | | 10.4.1 | | 15.2.8 | | 22.3 | |
| 4.5.2 | | 8.4.2 | | 10.4.2 | 05/04/84 | 15.3.1 | | 23 | |
| 5.2.1 | | 8.4.3 | | 10.4.3 | 05/04/84 | 15.3.2 | | | |