

0	1	7	8	A	R	A	N	0	2	12	14	0	0	-	0	0	0	0	0	0	-	0	0	0	3	25	4	1	1	1	1	1	4	30	57	CAT	58	15
				LICENSEE CODE										LICENSE NUMBER										LICENSE TYPE														
0	1	7	8	REPORT SOURCE				16	60	0	5	1	0	0	0	3	6	8	17	68	1	2	1	1	1	8	2	18	74	0	5	1	1	5	8	4	19	80
				DOCKET NUMBER								EVENT DATE								REPORT DATE																		

EVENT DESCRIPTION AND PROBABLY CONSEQUENCES 10

Approximately one hour after a reactor and turbine trip, and following emergency feedwater (EFW) actuation, the discharge control valve from the emergency feedwater pump 2P-78 to the 'B' steam generator (2CV-1075), failed to respond (shut) when manual control of EFW was attempted. The equipment required for the redundant flow path remained operable. Other LERs concerning Unit 2 EFW valves include 50-368/82-025, 82-019, 82-018, 82-015, 81-032, 80-008, 79-092, 79-090, 79-089, 79-088, 79-068, 79-054, 79-051, 79-043, 79-040, 79-037, 79-036, 79-035, 79-028, 78-026, 78-009. This occurrence is reportable per Technical Specification 6.9.1.9.b.

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP SUBCODE		VALVE SUBCODE		REVISION NO		
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17
0	9	H	H	E		A		V	A	L	V	O	P	A	Z	
7	8	9	10	11	12	13		14				15		16		17
LER/RO		EVENT YEAR		SEQUENTIAL		OCCURRENCE				REPORT		REVISION				
REPORT		18		REPORT NO.		CODE				TYPE		NO				
NUMBER		21		23		24				27		28				
		22				26				29		31				

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
1 B 118	1 A 119	1 Z 120	1 Z 121	1 0 1 0 1 0 1 0 1 22	1 N 123	1 Y 124	1 A 125	1 B 1 3 1 5 1 0 1 26
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 | 0 | The cause of this event appears to have been a combination of loose leads in the motor and low hydraulic
1 | 1 | pressure due to air in the hydraulic oil. Immediate action was to tighten the leads in the motor and to
1 | 2 | replace the hydraulic oil. These Weston-Hydraulic valve actuators were replaced with Limitorque SMB-000-2
1 | 3 | 460V A.C. motor operators during the 2R3 refueling outage.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	6	128	10	12	13	30	31	132
8	9	10	12	13	44	45	46	80	81
				NA		A		Operator Observation	

ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	2	33	NA	35	NA	36
8	9	10	11	44	45	46	80

PERSONNEL EXPOSURES										44	45	80
NUMBER		TYPE		DESCRIPTION								
1	7	0	0	0	37	2	38	NA				
8	9	10	11	12	13							

PERSONNEL INJURIES										80
NUMBER				DESCRIPTION						
1	8	0	0	0	40	NA				
8	9	11	12							41
LOSS OF OR DAMAGE TO FACILITY										80

LOSS OF OR DAMAGE TO FACILITY	
TYPE	DESCRIPTION
1 9	1 Z 42 NA

PUBLICITY										
ISSUED		DESCRIPTION								
2	0	1	N	144	NA					145
9					10					68
										69

NAME OF PREPARER Phillip B. Wade

PHONE: (501) 964-3100

8405240303 840515
PDR ADOCK 05000368
S PDR

IE 22



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May 15, 1984

2CAN058407

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 82-047/03X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.b, attached is the subject report concerning failure of the emergency feedwater pump discharge control valve to close. This is an update to a previous submittal dated January 11, 1983.

Very truly yours,

John R. Marshall
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard P. Denise, Director
Division of Resident Reactor Projects
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U. S. Nuclear Regulatory Commission
Region IV
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IE22
1/1