NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
North Anna Unit 2	0 5 0 0 0 3 3 9	1 OF 0 2

TITLE (4)

Manual Reactor Trip From 2% Power Due To Loss Of Normal Feedwater

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	0 5	8 4	8 4	0 0 2	0 0	0 5	1 7	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9): 1

POWER LEVEL (10): 0 1 2

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
E. Wayne Harrell	AREA CODE 7 0 3 — 8 9 4 - 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	A A	I M O D	W 1 2 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 5, 1984, during a scheduled rampdown for reactor trip breaker maintenance, Unit 2 was manually tripped from 2% power after receiving an automatic turbine trip signal and a loss of normal feedwater. The turbine trip signal was due to high water level in the 5B feedwater heater. A hi-hi level in a 5th point feedwater heater causes a trip of the main condensate and feedwater pumps and opens the condenser vacuum breaker. The reactor did not receive an automatic reactor trip since the unit was below the P-10 setpoint of 10% power.

All automatic reactor features functioned normally. The IRPI for control rod H-14 indicated 25 steps after the trip. The reactor was emergency borated for at least 7 minutes due to the apparent stuck rod.

The unit was stable in mode 3 immediately after the trip. The 5B feedwater heater level was only high for a few seconds. Once the level dropped back below the hi-hi setpoint, normal feedwater was restored and the auxiliary feedwater pumps were secured.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
North Anna Unit 2	0 5 0 0 0 3 3 9	8 4	— 0 0 2	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On May 5, 1984, during a scheduled rampdown for reactor trip breaker maintenance, Unit 2 was manually tripped from 2% power after receiving an automatic turbine trip signal and a loss of normal feedwater. The turbine trip signal was due to high water level in the 5B feedwater heater. A hi-hi level in a 5th point feedwater heater causes a trip of the main condensate and feedwater pumps and opens the condenser vacuum breaker. The reactor did not receive an automatic reactor trip since the unit was below the P-10 setpoint of 10% power. The 5B feedwater heater was later determined to have leaking tubes which were plugged.

All automatic reactor features functioned normally. The IRPI for control rod H-14 indicated 25 steps after the trip. The reactor was emergency borated for at least 7 minutes due to the apparent stuck rod. The IRPI module was found to be out of calibration. The module was recalibrated and the channel returned to service. The auxiliary feedwater pumps all started on loss of normal feedwater pumps and maintained the steam generator water levels.

The unit was stable in mode 3 immediately after the trip. The 5B feedwater heater level was only high for a few seconds. Once the level dropped back below the hi-hi setpoint, normal feedwater was restored and the auxiliary feedwater pumps were secured.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

May 17, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No. N-84-006
NO/DAH: 11
Docket No. 50-339

License No. NPF-7

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No. LER 84-002-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

IE22
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