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May 18, 1984

Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Crutchfield:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
SEP Topic No. XV-19 LOCA Resulting From
Spectrum of Postulated Piping Breaks
Within the Reactor Coolant Pressure Boundary

During the review of the subject SEP topic, the NRC staff performed an independent analysis on the radiological consequences of a design-basis loss-of-coolant accident. The analysis resulted in excessive thyroid doses for the 30 day low population zone boundary (341 rems). In view of the small difference between the calculated offsite dose and the acceptance criteria (10CFR100.11) and the conservatism in the staff's calculational techniques, the NRC staff concluded that further analysis of this event is not warranted.

However, a significant contribution to the calculated offsite dose was from the main steam line isolation valve (MSIV) leakage. On the basis of previous experience of MSIV leakage at Oyster Creek Nuclear Generating Station, the NRC staff concluded that GPU Nuclear should develop and implement a preventive maintenance program aimed at maintaining the MSIV leakage within the existing limit, or justify why the existing maintenance program is adequate.

In September, 1982 BWR Owners Group formed a subcommittee to address the MSIV-leakage issue with investigation in the following areas:

- (1) Development of a technique for evaluation of BWR MSIV leakage contribution to radiological dose rate.

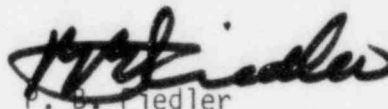
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- (2) Collection of MSIV leakage data to establish clear cause for local leak rate test failure.
- (3) Development of potential operator-action guidelines for preventing MSIV leakage or mitigating its effects.

GPU Nuclear has been an active participant in the Subcommittee. The investigation has recently been completed and the final reports addressing the areas of concern listed above have been issued to the member utilities of the Subcommittee. GPU Nuclear will shortly be evaluating the reports for plant specific applications. The results of this evaluation and identification of any related specific actions to be taken for Oyster Creek will be transmitted to you by the end of 1984.

Very truly yours,



P. B. Friedler
Vice President and Director
Oyster Creek

1r/0235e

cc: Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
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