

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Calvert Cliffs Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 3 1 1 8

PAGE (3)

1 OF 0 3

TITLE (4)

MAIN STEAM SAFETY VALVE SETPOINTS OUT OF TOLERANCE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
0	4	2	1	8	4	8	4	0	0	4	0	0	0	5	1	8	8	4	N/A	0 5 0 0 0 0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																	
3			20.402(b)			20.406(a)			80.73(a)(2)(iv)			73.71(b)								
POWER LEVEL (10)			20.406(a)(1)(i)			80.38(a)(1)			X 80.73(a)(2)(v)			73.71(a)								
0 1 0 0			20.406(a)(1)(ii)			80.38(a)(2)			80.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 388A)								
			20.406(a)(1)(iii)			80.73(a)(2)(i)			80.73(a)(2)(viii)(A)											
			20.406(a)(1)(iv)			X 80.73(a)(2)(ii)			80.73(a)(2)(viii)(B)											
			20.406(a)(1)(v)			80.73(a)(2)(iii)			80.73(a)(2)(ix)											

LICENSEE CONTACT FOR THIS LER (12)

NAME

C. R. Mahon, Senior Engineer - PMD

TELEPHONE NUMBER

AREA CODE

3 0 1 1 2 6 1 0 1 4 1 8 5 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	SIB	IRIV	D121413	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 21, 1984 the plant was shutdown in MODE 3 in preparation for a refueling outage. The reactor coolant temperature was lowered to approximately 500 F. In that condition, a surveillance test procedure was performed to check and adjust as necessary the Main Steam Safety Valve (MSSVS) Setpoints. Thirteen of sixteen MSSVS were found to be out of tolerance as specified in the ASME Boiler and Pressure Vessel Code Section III. The necessary MSSVS were placed back into tolerance prior to completion of the test. At the conclusion of the test the plant was placed into cold shutdown in order to continue with the refueling outage. To prevent recurrence of this event the MSSVS have been placed on a periodic maintenance schedule and the frequency of setpoint testing will be increased.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Calvert Cliffs Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 8 8 4 - 0 0 4 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 368A's) (17)

On April 21, 1984, thirteen of sixteen main steam safety valves (SB-RV) were found out of their setpoint tolerance band. The plant was in MODE 3 and reactor coolant (AB) temperature was approximately 500 (F) at the beginning of a refueling outage. The main steam safety valves were found out of tolerance during a surveillance test procedure. As required by the surveillance test, the main steam safety valves were adjusted back into their setpoint tolerance band by 1900 on April 21, 1984. The plant was cooled down and entered MODE 4 at 0240 on April 22, 1984. The plant entered MODE 5 at 1105 on April 22, 1984 and subsequently continued with the refueling outage.

The plant contains two Steam Generators (SG) each with its own steam header (SB). Each of the main steam headers contain eight main steam safety valves. The main steam safety valves are a Dresser Industries type C-3707RA maxiflow safety valve, 1500 PSIG class; constructed in accordance with ASME Boiler and Pressure Vessel Code Section III.

The relieving capacity of the main steam safety valves is sized to keep secondary system pressure limited to its design pressure during the most severe anticipated system operational transient. The most severe anticipated operational transient is a main turbine trip from 100 percent power coincident with an assumed loss of condenser (SQ) heat sink and no turbine bypass (SO) to the condenser. At the time of the event the plant was in MODE 3 with both Steam Generators and condenser operable. The technical specification basis for 3/4.7.1.1 states in part that two operable main steam safety valves are required to remove decay heat. Three valves were within tolerance at the time of the event. In assessing the event under other reasonable conditions a turbine trip from one hundred percent power was considered. The as found status of the main steam safety valves is shown in the below table:

Setpoint	No. of Valves that should be at that setpoint	No. of valves as found at that setpoint
985	4	2
995	4	1
1015	4	5
1035	4	2
Below all setpoint bands	0	4
Above all setpoint bands (Max 1065)	0	2

As can be seen from the table, only two valves were above any of the setpoint bands. The Nuclear Steam Supply System vendor for the plant has analyzed the as found setpoints and ascertained that the plant was not operating in an unsafe condition, the safety analysis was not violated, nor would offsite dose limits have exceeded the limiting offsite dose event (SLB).

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Calvert Cliffs Unit 2	05000318	84	-004	-00	03	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Past corrective actions have been to open and inspect main steam safety valves that were out of tolerance or that had slight seat leakage. No corrective maintenance other than seat polishing has been necessary. To prevent future recurrence of this event the main steam safety valves have been placed on a periodic internals inspection. The frequency of setpoint testing will be increased such that more setpoints will be checked on a refueling outage cycle basis than required by the inservice valve testing program. This should prevent any setpoint drift. Three main steam safety valves have been selected for overhaul during the current refueling outage to ascertain if any unique problems exist. Past performance of these valves has shown that since October 1979 fifteen tests have been conducted on individual main steam safety valves on Unit 2. Only one main steam safety valve was found out of tolerance until the testing of this event.

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

May 18, 1984

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-318
License No. DPR 53

Dear Sir:

The attached LER 84-004 is being sent to you as required by
10 CFR 50.73.

Should you have any questions regarding this report, we would be
pleased to discuss them with you.

Very truly yours,

LBR

L. B. Russell
Plant Superintendent

cfm
LBR:CFM:srm

cc: Dr. Thomas E. Murley
Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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