

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Nine Mile Point Unit #1DOCKET NUMBER (2)  
0 5 0 0 0 2 2 0PAGE (3)  
1 OF 3TITLE (4)  
Failure of Reactor Head Safety ValvesEVENT DATE (5)  
MONTH DAY YEAR  
0 4 1 2 8 4 8 4  
LER NUMBER (6)  
YEAR SEQUENTIAL NUMBER REVISION NUMBER  
0 0 3 0 0  
REPORT DATE (7)  
MONTH DAY YEAR  
0 5 1 2 8 4  
OTHER FACILITIES INVOLVED (8)  
FACILITY NAMES  
DOCKET NUMBER(S)  
0 5 0 0 0 0OPERATING MODE (9)  
POWER LEVEL (10)  
0 0 1 0  
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)  
20.402(b)  
20.406(a)(1)(i)  
20.406(a)(1)(ii)  
20.406(a)(1)(iii)  
20.406(a)(1)(iv)  
20.406(a)(1)(v)  
20.406(a)  
20.38(a)(1)  
20.38(a)(2)  
20.73(a)(2)(i)  
20.73(a)(2)(ii)  
20.73(a)(2)(iii)  
20.73(a)(2)(iv)  
20.73(a)(2)(v)  
20.73(a)(2)(vi)  
20.73(a)(2)(vii)(A)  
20.73(a)(2)(vii)(B)  
20.73(a)(2)(ix)  
73.71(b)  
73.71(c)  
OTHER (Specify in Abstract below and in Text, NRC Form 366A)LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Kim Dahlberg, Site Maintenance Superintendent - Nuclear  
TELEPHONE NUMBER  
AREA CODE  
3 1 5 3 4 9 - 2 4 4 3COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)  
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC  
X S B R V D 2 4 3 YSUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

## ABSTRACT

During the current refueling outage, problems with the testing of Dresser Safety Valves were encountered which resulted in the inability of the valves to be popped within  $\pm 1\%$  of set pressure. The cause of the failures is undetermined but test laboratory personnel suggest that the failures are due to normal wear and tear on the valves during their service lives. In total, 12 of the 16 Reactor Head Safety Valves were found to be outside the  $\pm 1\%$  tolerance.

Corrective action taken was to disassemble, and repair the valves.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Nine Mile Point Unit #1	DOCKET NUMBER (2)  0 5 0 0 0 2 2 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 0 3	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 386A's) (17)

NARRATIVE

On April 6-8, 26, 29, 1984, during the current refueling outage, 12 of the 16 Reactor Head Safety Valves failed to exhibit a set pressure within  $\pm 1\%$  of the Tech Spec set pressure.

The valves, manufactured by Dresser Industries, were removed from the Reactor Head and transferred off-site to be tested at Wyle Laboratories.

The following is a test summary of the applicable valves:

<u>Test Date</u>	<u>Serial No.</u>	<u>Tech Spec Set Pressure (psig)</u>	<u>As Found Set Pressure (psig)</u>	<u>Deviation from Tech Spec Set Pressure (%)</u>
4/6/84	BK6316	1245	1295	+4.02
4/6/84	BK6255	1236	1261	+2.02
4/6/84	BK6313	1245	1196	-3.94
4/7/84	BK6303	1254	1215	-3.91
4/7/84	BK6280	1236	1216	-1.62
4/7/84	BK6521	1218	1250	+2.63
4/8/84	BK6522	1218	1197	-1.72
4/8/84	BK6297	1245	1219	-2.09
4/8/84	BK6301	1254	1298	+3.51
4/8/84	BK6524	1218	1205	-1.07
4/26/84	BK6254	1254	1279	+1.99
4/29/84	BK6267	1236	1253	+1.38

The suggested cause of this event is that the main springs and spring washers on the valves were worn due to normal service operation.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
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Nine Mile Point Unit #1	0 5 0 0 0 2 2 0	8 4	— 0 0 3	— 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

ASSESSMENT OF SAFETY CONSEQUENCES

With 12 of 16 safety valves outside the  $\pm 1\%$  tech spec tolerance, the transient analysis which proves that the reactor will not exceed the safety limit of 1375 psig in the worst case event is potentially invalid. However, 6 of the 12 valves were set too low, so the total plus and minus percentages nearly equal out, (ie, the errors are compensating). Since the average deviation for the 12 valves is  $+0.1\%$ , which is within an average  $\pm 1\%$ , the total pressure relieving capability is within the bounds of the capability assumed in the transient analysis. Therefore, the safety limit of 1375 psig would not have been exceeded.

CORRECTIVE ACTIONS

The 12 valves were disassembled and repaired. All main springs and spring washers have been replaced and the valves retested satisfactory.

## NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST  
SYRACUSE, N. Y. 13202

May 14, 1984

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555RE: Docket No. 50-220  
LER 84-03

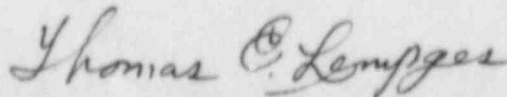
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following  
Licensee Event Report:

LER 84-03      Which is being submitted in accordance with  
10 CFR 50.73 (a)(2)(ii), "Any event or condition  
that resulted in the condition of the Nuclear  
Power Plant, including its principal safety  
barriers, being seriously degraded"

The report was completed in the format designated in NUREG-1022,  
dated September 1983.

Very truly yours,

Thomas E. Lempges  
Vice President  
Nuclear GenerationTEL/lo  
Attachment (3 copies)  
cc: Dr. Thomas E. Murley  
Regional AdministratorIE22  
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