

MONTHLY OPERATIONS REPORT

JUNE 1984

Throughout the report period, Oyster Creek Station remained shutdown for current Maintenance and Refueling.

Replacement of the IRMs/SRMs started and continued throughout the report period. To date, IRMs 12, 14, 15, 16 and 18 have been completed. The major problem experienced with the task is removal of the dry tubes. The IRM/SRM dry tubes are the originally installed equipment. Over the years, crud has collected in the in-core housing and this is thought to be the major contributor of interference problems experienced during dry tube removal. Therefore, cutting of the dry tubes prior to removal is necessary due to the inability to remove the dry tube by "detenting" the upper grid plunger. In addition, a considerable amount of time was spent on the removal of SRM 22. Special tooling had to be fabricated before it could be removed. Furthermore, a positive seal on the replacement dry tube back seat could not be initially obtained. The new dry tube was successfully replaced on July 5, 1984.

The top section of IRM 18 (approximately 18 inches), including the plunger assembly, was discovered missing when this tube was removed (IRM 18 was one of the IRMs which had a 360° circumferential crack in its dry tube). The missing piece was located on a nearby fuel support casting. The spring for the plunger assembly was removed, but the remaining section remained on the fuel support casting and will be removed at the first opportunity. Replacement of the remaining IRMs/SRMs is the priority task. In addition to this work, LPRMs 36-33 and 36-17 were replaced during this report period.

Diesel Generator No. 2 was returned to an operable status on June 5, 1984. The Diesel Generator was out of service due to a governor actuator problem.

At the end of the report period, the torus was filled and functional testing of Core Spray System I was in progress. When completed, Core Spray System II will be released for pump impeller lock nut inspection.

Fire Diesel Pump 1-1 was inoperable for an extended period of time (June 20 to July 1). The fire pump was initially taken out of service for vendor inspection. However, upon testing, the pump did not meet acceptance criteria for discharge pressure. The problem was traced to a faulty pump auto-vent valve which was repaired.

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Dilution pump No. 2 ran satisfactorily during its 24-hour test run using the recently installed 90 micron filters with the Fire Water System supplying flow to the pump seals. The system is still unable to operate without use of the Fire Water System. Additional meetings are being held with the various departments involved in the project to develop a course of action to resolve the problems.

At the end of the report period the circulating water system tunnel was being prepared for flooding. The circulating water system is required to support the RBCCW System outage scheduled for early July.

The Operations Department is venting, timing and performing interference checks on selected CRD's as plant conditions permit.

Testing of the plants' electrical power distribution cables in the turbine building continued throughout the report period. Replacement of the cable run to "1C" feedwater pump is in progress.

Weld repairs to No. 1-1 RBCCW heat exchanger were completed. Also, a small leak was identified and repaired (tube plugged) on "A" fuel pool heat exchanger. It is suspected that the leak was such that the tube leaked only during normal operation (when higher differential pressure across the bundle was established) thus making it difficult to detect during test conditions.

Air compressor 1-1 was taken out of operation twice during the report period; once for an oil change and once due to an oil leak. Air compressor No. 1-2 was out of service twice during the report period; once for an oil change and once due to an unloading problem. M&C was working on the unloading problem at the end of the report period. Air compressor No. 1-3 was removed from service four different times during the report period to support instrument calibration, preventive maintenance and to repair oil leaks.

"B" liquid poison pump was declared inoperable for a seven day period due to excess vibration. Plant Engineering was requested to take vibration readings and evaluate the data. It was determined that the pump was not vibrating excessively and was returned to service.

Weld repairs to the isolation condenser piping continued throughout the report period.

The HFA relay fogging problem is still being investigated by Plant Engineering. Testing/analysis of the relays is in progress by various company departments and the relay vendor (GE). GE will be providing a test procedure suitable for testing the presently installed relays.

The following Licensee Event Reports were submitted during June 1984:

Licensee Event Report No. 50-219/84-008

While performing Local Power Range Monitor (LPRM) replacement work during the outage, operators visually noticed that the dry tube associated with Intermediate Range Monitor (IRM) 12 appeared to be bent near the upper core grid. An underwater TV camera inspection performed on the dry tube in February 1984 showed a significant amount of cracking in the top portion of tube. Detailed inspection conducted by the QA Department revealed that a total of seven (7) IRM and one (1) SRM dry tubes were cracked. Replacement of these dry tubes will be completed prior to start-up from the present refueling outage.

Licensee Event Report No. 50-219/84-010

For an undetermined number of iterations the fuel pool gates may have been moved over irradiated fuel bundles in the fuel pool. This violates the Technical Specifications requiring that no object in excess of the weight of one fuel assembly (approximately 485 lbs.) be moved over stored irradiated fuel. The handling procedure for the fuel pool gates will be revised to prevent lifting the gates above irradiated fuel.

Licensee Event Report No. 50-219/84-011

Both trains of the Standby Gas Treatment System were rendered inoperable for nine (9) minutes while performing preventive maintenance on a circuit breaker. A modification is in progress to eliminate the problem of the inlet and outlet valves for both trains failing open on loss of air when the circuit breaker is racked out for maintenance.

Licensee Event Report No. 50-219/84-014

The test of the Reactor Low Level Instrumentation was not performed within the time period required by Technical Specifications. The event is attributed to personnel error. The involved personnel have been instructed on their responsibilities as outlined in the procedure which implements the surveillance test program.

Licensee Event Report No. 50-219/84-013

Post indicating valve V-9-12 which branches off the fourteen inch Fire Water Main was damaged by a maintenance vehicle. This resulted in a loss of the Fire Suppression Water System. Immediate actions consisted of isolating the line which contained the damaged valve, testing the redundant Fire Protection Water System for operability, and aligning the redundant system to supply the underground loop which feeds the Fire Suppression Water System. As a result of this incident all post indicating valves in similar high traffic areas were inspected for adequate physical protection.

Licensee Event Report No. 50-219/84-12

During a scheduled load test on Emergency Diesel Generator No. 1 (EDG-1), a diesel fuel oil day tank low level alarm for unit one was received in the Control Room. Subsequent investigation revealed that the diesel fuel oil transfer pump control switch for EDG-1 was in the OFF position. In the OFF position, fuel oil is not automatically transferred to the diesel day tank from the main fuel storage tank. This resulted in EDG-1 being considered inoperable. Since EDG-2 was out of service for governor repairs, both Emergency Diesel Generators were simultaneously inoperable. Future solutions to be implemented include incorporation of a check of the transfer pump control switch position into the plant tour sheets and installing a keylock control on the transfer pump control switch for each Emergency Diesel Generator to prevent repositioning without the Group Shift Supervisor's concurrence.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 1984

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE 7/9/84
 COMPLETED BY R. Baran
 TELEPHONE 971-4640

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	12120	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: June, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: N/A

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	720.0	4368.0	127296.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0	0	136224729
18. GROSS ELEC ENER (MWH)	0	0	46056905
19. NET ELEC ENER (MWH)	-2250	-11982	44273701
20. UT SERVICE FACTOR	0.0	0.0	65.0
21. UT AVAIL FACTOR	0.0	0.0	65.0
22. UT CAP FACTOR (MDC NET)	0.0	-0.4	56.1
23. UT CAP FACTOR (DER NET)	0.0	-0.4	53.5
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	8/7/84		

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET # 50-219
UNIT. Oyster Creek #1
REPORT DATE JULY 03, 1984
COMPILED BY DONALD V. NOTIGAN
TELEPHONE # 609-971-4695

MONTH JUNE, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0

REFUELING INFORMATION - June, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: 8/07/84

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

NONE

Scheduled date(s) for submitting proposed licensing action and supporting information:

1. Amendment #7 to NEDO-14195: G.E. Reload Analysis incorporating LOCA analysis. (5/30/84)
2. Technical Specification Change Request 119 for Scram Discharge Instrument Volume System Modification. (5/15/84)
3. Submittal for SEP Topic "ECCS Actuation" dated 6/4/84.
4. Submittal for maintenance dredging of the Circulating Water Intake Structure dated 6/4/84.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 503
(b) in the spent fuel storage pool = 1037

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800

Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.



GPU Nuclear Corporation

Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

July 16, 1984

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555


Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731