

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 7 3					PAGE (3) 1 OF 1	
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TITLE (4) Reactor Water Cleanup Differential Flow Isolation																
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EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0 6	2 4	8 4	8 4	0 3	3	0 0	0 7	1 8	8 4					0 5 0 0 0			

OPERATING MODE (9) 3		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)														
POWER LEVEL (10) 0 0 0		20.402(b)			20.406(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)					
		20.406(a)(1)(i)			50.38(c)(1)			<input type="checkbox"/> 50.73(a)(2)(v)			73.71(c)					
		20.406(a)(1)(ii)			50.38(c)(2)			<input type="checkbox"/> 50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.406(a)(1)(iii)			50.73(a)(2)(i)			<input type="checkbox"/> 50.73(a)(2)(vii)(A)								
		20.406(a)(1)(iv)			50.73(a)(2)(ii)			<input type="checkbox"/> 50.73(a)(2)(viii)(B)								
		20.406(a)(1)(v)			50.73(a)(2)(iii)			<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER						
NAME Daniel R. Smythe, extension 292										AREA CODE 8 1 5 3 5 7 - 6 7 6 1						

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	
X	J M Z	9 9 9	Z 9 9 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 6/24/84, at 1730 with Unit 1 in Hot Shutdown at 620 psig an isolation of the Reactor Water Cleanup system (RWC, CE) occurred due to high differential flow. The RWC system was operating in the Blowdown Mode at the time of the event. No system leaks could be found indicating the differential flow isolation was caused by differences in water temperature (density) at the inlet and outlet portions of the system. The RWC system was then placed back in operation with no further problems occurring that day.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

I. EVENT DESCRIPTION

On 6/24/84 at 1730 the Reactor Water Cleanup system (RWCU, CE) isolated on high differential flow. Unit 1 reactor was in Hot Shutdown (Mode 3) at 620 psig. At the time of the event the Reactor Water Cleanup system was operating in the Blowdown Mode with about 75 gpm being rejected from the reactor to the main condenser (SG).

II. CAUSE

During normal Reactor Water Cleanup system operation, differential flow is indicated with no leaks present. This is due to differences in water temperature (density) of inlet and outlet portions of the system and temperatures used as a basis for flow instrumentation calibration. During startup when the Reactor Water Cleanup system is operating in the Blowdown Mode, indicated differential flow increases due to the system operating at less than rated temperature and blowdown flow being at a temperature well below system inlet and outlet to feedwater flow temperatures. In this case, the indicated differential flow was just enough to initiate the system isolation.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The RWCU system isolated per design. Since the RWCU system was immediately placed back in service, plant operation was not affected.

IV. CORRECTIVE ACTION

Immediately after the RWCU system isolation, the system was placed back in service with no problems encountered.

Applicable procedures are being reviewed for possible revision to alert Operators that this can occur during plant startup, and to give guidance on actions which can be taken to reduce the likelihood of isolations of RWCU occurring on differential flow. (AIR 01-84-131)

Also, a review of the design temperatures used as a basis of flow instrumentation calibration is being conducted for possible revision to more accurately reflect system operating temperatures. AIR 01-84-67106 is written to track this review.

V. PREVIOUS OCCURRENCES

There have been three previous occurrences of a similar nature (i.e., differential flow isolation of the RWCU system due to differences in water temperature in various points of the system). These occurrences were described in LER's 373/84-030-00, 374/84-023-00 and 374/84-029-00.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Daniel R. Smythe, (815)357-6761, extension 292.



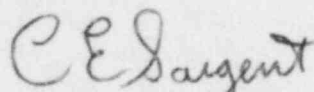
Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

July 18, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-033-00, Docket #050-373 is being submitted to your office in accordance with 10 CFR 50.73.


G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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