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SENIOR VICE PRESIDENT
NUCLEAR

July 16, 1984
BECO 84-106

Mr. Thomas E. Murley
Office of Inspection and Enforcement
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

License No. DPR-35
Docket No. 50-293

Report of Changes, Tests, and Experiments
Performed at Pilgrim Nuclear Power Station

Dear Sir:

In accordance with 10CFR50.59(b), Boston Edison hereby submits a report containing a brief description and safety evaluation for the changes, tests and experiments performed at Pilgrim Nuclear Power Station for the period of January 22, 1983 thru January 21, 1984.

Attachment 1 contains a brief description and a summary of the safety evaluations for these changes and tests which did not:

- increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety,
- create a possibility for an accident or malfunction of a different type than any evaluated previously in the FSAR; or
- reduce the margin of safety as defined in the basis for any technical specification.

We trust that this will be acceptable. However, should you have any questions or concerns, please do not hesitate to contact us.

Very truly yours,

WJ Harrington

ERM/kmc

Attachments: 1 signed original and 12 copies

cc: Director,
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
c/o Distribution Service Board, JCC, ADM
Washington, D. C. 20555

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ATTACHMENT 1
Plant Changes
(Plant Design Change Request - PDCR)

PDCR 76-80A South Access Roadway and Warehouse Modifications

This PDCR provided reconstruction of the south end of the warehouse to provide vehicle access to the Protected Area via a new roadway from the new employees parking lot.

Ref. FSAR Figure 1.6-1.

Safety Evaluation No. 76-80.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 80-17 Post-Accident Sampling System

This PDCR provided the capability of obtaining representative liquid and gas samples from within Primary Containment for radiological and/or chemical analysis in association with a possible Loss of Coolant Accident (LOCA).

Ref. FSAR Section 10.19.
Figure 10.19-1.

Safety Evaluation No. 849.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 80-37 Modifications to Support Hot Machine Shop Mezzanine Construction

This PDCR involved rerouting and relocating piping, pipe components, ducts and duct components and equipment to support the installation of a mezzanine above the Hot Shop.

Ref. FSAR Figure 12.1-13.

Safety Evaluation No. 1065.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-24

Installation of Sample Connections to SSW Discharge Lines

This PDCR involved removal of a blind flange connection and the addition of a new flange, nipple and gate valve to ensure that the chlorination system is capable of maintaining equal chlorine concentration in both SSW discharge headers.

Ref. FSAR Section 10.7.1

Safety Evaluation No. 1132.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-48

RCIC Automatic Restart Modifications

This PDCR involved modifying the initiation logic of the RCIC system so that it will automatically restart on low water level following a high reactor level trip.

Ref. FSAR Section 4.7.5

Figures 4.7-2, 4, 5, 6

Safety Evaluation No. 1194 and 1359.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-38

Replacement of HPCI/RCIC Self-Contained Type Pressure Regulating Valves

This PDCR involved replacement of the HPCI/RCIC pressure regulating valves with similar self-contained type pressure regulating valves to decrease the probability of the valves failing open on loss of Station air. This change also involved relocating the restricting orifice upstream of the gland seal condenser.

Ref. FSAR Figures 4.7-2, 6.4-1

Safety Evaluation No. 1172.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 82-17B Seawater Delta Temperature

This PDCR modified the measurement of Condenser Delta T with the installation of temperature sensors in the intake and discharge canals with transmitters to the plant computer.

Ref. FSAR Section 11.6
Figure 11.6-1

Safety Evaluation No. 1472.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 82-18 Piping Modification to Reroute Cleanup Sludge

This PDCR involved the installation of a 2" carbon steel line between the sludge pump discharge, from the reactor cleanup sludge receiving tank, to the radwaste trucklock filtering/demineralizing container.

Ref. FSAR Section 9.3.4.2.1, 9.3.4.2.2
Figures 9.2-4, 9.3-1

Safety Evaluation Nos. 1438, 1442, and 1469.

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

Facility Changes

New Administration/Service Building

This CR added a new Administration/Service building to the site environs to serve as an office facility for administrative and clerical personnel. The complex also includes a warehouse, laboratories, and shops.

Ref. FSAR Sections 1.6.1.2, 12.2.2.5.2
Figure 1.6.1

Safety Evaluation - not applicable. This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

Trash Compaction Facility

This CR added a Trash Compaction Facility to the site environs to process both contaminated and non-contaminated dry compactable waste generated from normal operation conditions.

Ref. FSAR Section 9.5
Figure 1.6-1

Safety Evaluation - not applicable. This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

Plant Tests
(Temporary Procedures - TP)

- TP-82-23 To perform operability testing of 1001-68 A&B for ASME Section XI testing when RHR System is placed in shutdown cooling mode, one loop at a time.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

- TP-82-43 To preclude egress of resins from Pilgrim's Process Building to environments. For Alara purposes, cleaning will be performed simultaneously with inspection. To judge performance of maintenance efforts, quantification and results will be reported and forwarded to the Chief Technical Engineer.

This test not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

- TP-82-80 To provide detailed instructions for station personnel to perform a test to determine possible effects on reactor water conductivity and main steam line radiation when CRD system suction is transferred from condensate feedwater header to the condensate storage tank.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

- TP-83-5 A test which will assist in diagnosing the movement problems being encountered with control rods as well as attempting to move the rods out to position 48.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

- TP-83-19 To provide post work testing proof of the CRDs scram outlet valve CV 302-127 operability while the drive is fully inserted, thereby lessening temperature transients to fuel. The Drive was successfully scrambled under MR 83-3-94. This test is to quantify time of CV opening as per GE Technical manual.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

- TP-83-22 To provide instructions for operating personnel to remove the first point heaters from service to increase power for end of cycle coastdown conditions.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-83-27 To provide detailed instructions to demonstrate that the Cable Spreading Room Fire Detection and Protection System will function as designed.

This test did not involve an unreviewed safety question as defined in 10 CFR 50.59(a) or a change to a technical specification.

TP-83-33 To provide instructions and administrative limitations to be used for partial or full isolation (feedwater side) and bypass of the fourth point "B" feedwater heater.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-83-55 To provide instructions and administrative limitations for partial bypass of the "B" low pressure feedwater heater string.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-83-60 To provide orientation, assure safe movement and storage of central rod blades within the fuel pool during the 1983 Refueling Outage.

This test did not involve an unreviewed safety question as defined in 10 CFR 50.59(a) or a change to a technical specification.