

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 1 7										PAGE (3) 1 OF 0 1 3																													
TITLE (4) Suppression Pool Level Outside Technical Specification Limits																																																	
EVENT DATE (5) MONTH DAY YEAR 0 1 2 8 8 4 8 4										LER NUMBER (6) SEQUENTIAL NUMBER REVISION NUMBER 0 0 8 0 1 0										REPORT DATE (7) MONTH DAY YEAR 7 1 9 8 4										OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S) 0 5 0 0 0																			
OPERATING MODE (9) 4										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																							
POWER LEVEL (10) 0 0 0										20.402(b) 20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)										20.406(c) 50.36(e)(1) 50.36(e)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii)										50.73(a)(2)(iv) 50.73(a)(2)(v) 50.73(a)(2)(vi) 50.73(a)(2)(vii)(A) 50.73(a)(2)(vii)(B) 50.73(a)(2)(viii)										73.71(b) 73.71(c) OTHER (Specify in Abstract Below and in Text, NRC Form 366A)									
LICENSEE CONTACT FOR THIS LER (12) NAME C.M. Powers, Reactor Engineering Supervisor																														TELEPHONE NUMBER AREA CODE 5 1 0 1 9 3 1 7 1 7 1 - 1 2 1 5 1 0 1 1																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2996																																																	
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC										CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC																																							
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SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) X NO																				EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR																													
ABSTRACT (16) (Type in 400 spaces or a space later; fifteen single-space typewritten lines) (16)																																																	
<p>Revision 0 to LER 008, issued on February 24, 1984, reported a failure of a suppression pool level indicator (CMS-LR/PR-4) and the subsequent determination that suppression pool level was below the Technical Specification limits.</p> <p>The purpose of this supplemental report is to address the loss of water in the suppression pool during the event due to an inadequate valve lineup. As previously reported, Technical Specifications (4.5.3.1) requires the suppression pool level to be maintained at a minimum of 30 feet 9-3/4 inches. Actual level was determined to be approximately 30 feet 5 inches and was verbally reported to the NRC pursuant to 10CFR50.72(2)(iii)(D).</p> <p>(It should be noted that the original report incorrectly stated that the Plant was in mode 5 at the time of the event. The Reactor vessel head was tensioned on January 26, 1984, two days preceding the event, and the mode switch was transferred from refueling to shutdown.)</p>																																																	
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 3/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039784	-	008	-	01	d 2 OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Operating Conditions

- a). Power Level - 0%  
b) Operating Mode - 4 (Prior to Initial Plant Heatup)

Event

During normal shutdown conditions prior to initial power operation, only one channel of suppression pool level indication was available. The suppression pool level recorder was inadvertently de-energized when the 120 VAC power supply breaker feeding it was opened at 1430 hours on January 27, 1984. The Control Room recorder was not noted as out of service until 1000 hours on January 28, 1984. The level decrease occurred during this 20 hour period. The NRC was notified at 1340 hours on January 28, 1984 pursuant to 10CFR50.72(2)(iii)(D).

When pool level indication was returned to service it was noted that condensate had accumulated in the reference leg of the transmitter causing an incorrect level indication in the Control Room. The cause of the violation was the result of a simultaneous loss of suppression pool level indication in the Control Room and the operation of the 'A' loop of the Residual Heat Removal (RHR) system in shutdown cooling while the 'B' loop was also lined up for shutdown cooling but was not in service. This configuration of the Residual Heat Removal system, as specified in the then existing plant procedure, provided a flow path to remove water from the wetwell through the inactive loop and inject it into the vessel where normal Reactor vessel level control actions discharged the water to the condenser hotwell. The flow path resulted from an incorrect cross-tie valve lineup which allowed suppression pool water to flow through the waterleg pump, RHR-P-3, to the discharge of RHR-P-2B back through the discharge check valve, RHR-V-31B, to the pump suction and into the A RHR system when RHR pump B shutdown cooling suction valve RHR-V-6B was left open. This lineup produced insufficient differential pressure to adequately backseat the RHR B discharge check valve.

Immediate Corrective Action

The Control Room indication was reenergized, a local measurement of the suppression pool level was taken, water was drained from the reference leg of the transmitter to provide accurate level indication and water level was restored in the suppression pool.

Further Corrective Action

The cause of the water loss was not identified until a similar event occurred approximately two weeks later. The corrective action taken after the recurrence was to close the Residual Heat Removal Pump suction cross-tie valve for the inactive loop when both loops are lined-up for shutdown cooling. A subsequent revision to the RHR system procedure was issued to specify the correct valve lineup. When not in the shutdown cooling mode, the Residual Heat Removal trains are maintained in their normal Low Pressure Coolant Injection mode lineup as required by Plant procedure.

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

To eliminate the level indication complication, the suppression pool level recorders were marked to show Technical Specification upper and lower level limits. A list of loads receiving power from each circuit breaker is being developed and the suppression pool level transmitter's reference legs are periodically drained.

## Safety Significance

There were no safety consequences as a result of this event. The reactor was in cold shutdown condition with the fuel having an insignificant fission product inventory.

## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

July 19, 1984

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 84-008-01

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-008-01 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and provides supplemental information to LER 84-008, Rev. 0.

Very truly yours,

*J. D. Martin*  
J. D. Martin (M/D 927M)  
WNP-2 Plant Manager

JDM:mm

Enclosure:  
Licensee Event Report No. 84-008-01

cc: Mr. John B. Martin, Administrator  
Region V, Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
1450 Maria Lane  
Walnut Creek, California 94596  
Mr. A. D. Toth, NRC Resident Inspector (901A)  
Ms. Dottie Sherman  
American Nuclear Insurers  
The Exchange Suite 245  
270 Farmington Ave.  
Farmington, CT 06032

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