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ANPP-24171-BSK/RQT

REGION V I&E

U. S. Nuclear Regulatory Commission
Region V
Creekside Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, CA 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-32
A 50.55(e) Potentially Reportable Deficiency Relating to
Socket Welds In Unit 1 Steam Generator Process Piping Are In
Violation of Requirements
File: 83-019-026; D.4.33.2

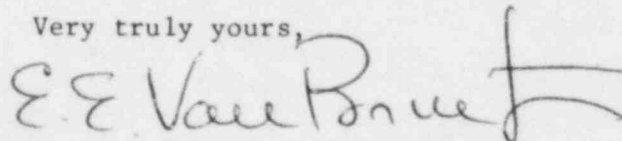
Reference: Telephone Conversation between T. Young and R. Tucker on
May 25, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the
referenced telephone conversation. At that time, it was estimated that a
determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by August 9, 1983, at which time a complete report will be
submitted.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President,
Nuclear Projects Management
ANPP Project Director

EEVB/RQT:ru

Enclosure

cc: See Page 2

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U. S. Nuclear Regulatory Commission
Page Two

cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

T. G. Woods, Jr.
G. C. Andognini
J. A. Roedel
D. B. Fasnacht
A. C. Rogers
B. S. Kaplan
W. E. Ide
J. Vorees
J. R. Bynum
P. P. Klute/D. D. Green
A. C. Gehr
W. J. Stubblefield
W. G. Bingham
R. L. Patterson
R. W. Welcher
R. M. Grant
D. R. Hawkinson
L. E. Vorderbrueggen
G. A. Fiorelli
S. R. Frost
J. Self

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway - Suite 1500
Atlanta, Georgia 30339

INTERIM REPORT - DER 83-32
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. Potential Problem

Specification 13-PM-204, Revision 11, paragraph 5.8.5 prohibits dissimilar metal socket weld connections in P8 to P1, P8 to P4, or P8 to P5 materials in ASME III Class 2 and 3 process piping when the operating temperature is 250 F or greater. During preparation for installation of Unit 2 valves 2PSGEV026 and 2PSGEV032, Field Engineering determined that the above requirement would be violated. An inspection of Unit 1 revealed that valves 1PSGEV026 and 1PSGEV032 had already been installed in violation of this requirement. Field Engineering issued a Field Change Request to correct this condition, and APS Quality Assurance subsequently issued APS CAR C83-72N to document this condition as a noncompliance.

The subject valves, located in the Steam Generator Wet Layup Recirculation lines (in containment), are Kerotest Model 329 2" globe stainless steel (P8 material) valves socket welded to 2" diameter carbon steel (P1 material) and alloy steel (P5 material) process piping. They are closed during normal plant operation, isolating the Quality Class S, Seismic Category III Wet Layup Recirculation pumps from the high temperatures and pressures of the Steam Generator blowdown lines (one valve and pump per blowdown line, i.e., per Steam Generator). These valves are otherwise used in conjunction with the Wet Layup Recirculation pumps to perform the following functions: 1) to furnish a continuous flow of water through the Steam Generators during shutdown periods; 2) to add chemicals during wet layup; and 3) to assist in draining the Steam Generators.

The subject condition is attributed to a failure to make the necessary changes in design drawings subsequent to agreement between Bechtel and Kerotest to substitute stainless steel valves for the originally specified Kerotest Model 327 alloy steel valves. Kerotest failed to qualify the seat weld for the Model 327 alloy steel valves and requested to supply stainless steel valves as a substitute. Bechtel Engineering authorized the substitution of stainless steel valves and, to comply with dissimilar metal socket weld requirements, indicated that the necessary transition pieces would be provided by Bechtel. This valve substitution was subsequently incorporated into the Material Requisition for Specification 13-PM-221C (Revision 4, issued 1/6/82) and the Valve Designation List (Revision 6, issued 3/29/82). However, the related piping isometric drawings, which are used by Construction, were never revised to show the required transition pieces or that the valves were stainless steel.

II. Approach To And Status Of Proposed Resolution

The subject valves are approximately 3'-9" from the Steam Generator blowdown lines (located between the Steam Generator blowdown lines and the Wet Layup Recirculation pumps) and will thus be subjected to temperatures exceeding 250 F during normal operation. The socket welds could therefore potentially fail due to the difference in thermal expansion and contraction of the dissimilar metals.

III. Projected Completion of Corrective Action and Submittal Of The Final Report

Evaluation of this condition and submittal of the Final Report is forecast to be completed by August 9, 1983.