

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2DOCKET NUMBER (2)
0 5 0 0 0 3 9 7 1 OF 0 2

TITLE (4)

Unscheduled Initiation of CR Emergency Filtration Unit

| EVENT DATE (8) | | | LER NUMBER (8) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | | | | | | | | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) | | | | | | | | | | | | |
| 0 | 6 | 2 | 0 | 8 | 4 | 8 | 4 | 0 | 6 | 7 | 0 | 0 | 0 | 7 | 1 | 2 | 8 | 4 | 0 | 5 | 0 | 0 | 0 |

| OPERATING MODE (9) | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11) | | | | | | | | | | | |
|--------------------|---|--|---|-------------------|--|-----------------|--|----------------------|--|--|--|----------|--|
| POWER LEVEL (10) | 0 | 0 | 1 | 20.402(b) | | 20.405(a) | | X | | 50.73(a)(2)(iv) | | 73.71(b) | |
| | | | | 20.405(a)(1)(i) | | 50.36(a)(1) | | 50.73(a)(2)(v) | | 73.71(a) | | | |
| | | | | 20.405(a)(1)(ii) | | 50.36(a)(2) | | 50.73(a)(2)(vi) | | X OTHER (Specify in Abstract below and in Text: NRC Form 366A) | | | |
| | | | | 20.405(a)(1)(iii) | | 50.73(a)(2)(i) | | 50.73(a)(2)(vii)(A) | | 50.72(b)(2)(ii) | | | |
| | | | | 20.405(a)(1)(iv) | | 50.73(a)(2)(ii) | | 50.73(a)(2)(viii)(B) | | | | | |
| 20.405(a)(1)(v) | | 50.73(a)(2)(iii) | | 50.73(a)(2)(ix) | | | | | | | | | |

LICENSEE CONTACT FOR THIS LER (12)
NAME
C.M. Powers, Reactor Engineering SupervisorTELEPHONE NUMBER
AREA CODE
5 0 1 9 3 1 7 7 - 1 2 5 0 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2996

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
|-------|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
| B | I | L | R | A | K | 0 | 2 | 0 | N |

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Control Room emergency filtration units (an ESF system) were automatically actuated due to a spike on the corresponding radiation monitoring system. The spike was associated with the operation of a reactor core isolation cooling valve.

After verifying that radiation levels were not above normal background, the radiation monitors and emergency filtration units were reset and returned to a normal lineup.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

| | | | | | | | |
|--|--|----------------|----------------------|--------------------|----------|----|-----|
| FACILITY NAME (1) Washington Nuclear Plant - Unit 2 | DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 4 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | 8 4 | 0 6 7 | 1 | 0 2 | OF | 0 2 |

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Plant Mode 2
b) Power Level < 1%

Event

A Control Room emergency filtration unit (WMA-FN-54A) was automatically actuated on 6-20-84 by a high-high radiation alarm originating from the outside air intake monitors (WOA-RIS-31A, 32A).

Immediate Corrective Action

Normal background radiation levels were observed at the monitors. The associated radiation recorder (WOA-RR-31) revealed that monitors 31A and 32A had received a spike of sufficient magnitude to trip the high-high radiation alarm. These alarms were promptly reset and the ESF system returned to normal. The spike originated from the operation of a reactor core isolation cooling valve (RCIC-V-45).

Notification was given to the NRC in accordance with the requirements of 10CFR50.72(b)(2)(ii).

Further Corrective Action

The process radiation monitoring system is highly susceptible to induced signal noise. The system installation is currently being redesigned to provide noise immunity.

Safety Significance

There were no safety consequences associated with this event and all plant systems performed as required.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

July 12, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

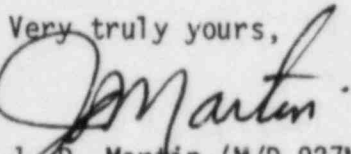
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-067

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-067 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1810 hours on June 20, 1984.

Very truly yours,



J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 84-067

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
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Walnut Creek, California 94596
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