

LICENSEE EVENT REPORT

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED INFORMATION)

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---------------------------------|--|--|--|--|--|--|--|--|--|--------------------------------|--|--------------------|--|---------------------------|--|--|--|--|--|-----------------------|--|--|--|-----------------------|------------------|--|--|--|-----------------------|--|
| LICENSEE NAME 01 I L D R S 2 | | | | | | | | | | LICENSE NUMBER 00-000000-00 | | | | | | | | | | LICENSE TYPE 41111 | | | | | EVENT TYPE 03 | | | | | |
| CATEGORY 01 CONT | | | | | | | | | | REPORT TYPE P O | | REPORT SOURCE L | | DOCKET NUMBER 050-0237 | | | | | | | | | | EVENT DATE 0110276 | | | | | REPORT DATE 012176 | |

EVENT DESCRIPTION

| | | |
|----|---|----|
| 02 | WHILE SHUTTING UNIT-2 DOWN FOR MAINTENANCE, A REACTOR | 80 |
| 03 | SCRAM OCCURRED ON LOW VACUUM AFTER CONDENSER VACUUM WAS | 80 |
| 04 | BROKEN. THE REACTOR WAS AT 181 PSIG WITH ALL RODS IN- | 80 |
| 05 | SERTED. THE SHUTDOWN COOLING SYSTEM WAS IN SERVICE | 80 |
| 06 | REMOVING DECAY HEAT. (50-237/1976-2) | 80 |

| | | | | | | | | | | | | | | | |
|-----------------------|--|-----------------|--|--------------------------|--|--|--|-------------------------------|--|----------------------------------|--|--|--|----------------|--|
| SYSTEM CODE 07 I A | | CAUSE CODE A | | COMPONENT CODE VALVEX | | | | PRIME COMPONENT SUPPLIER N | | COMPONENT MANUFACTURER W11615 | | | | VIOLATION N | |
|-----------------------|--|-----------------|--|--------------------------|--|--|--|-------------------------------|--|----------------------------------|--|--|--|----------------|--|

CAUSE DESCRIPTION

| | | |
|----|---|----|
| 08 | THE RACK ISOLATION VALVE FOR PRESSURE SWITCH 263-51B WAS | 80 |
| 09 | INADVERTENTLY LEFT CLOSED AFTER A PREVIOUS SURVEILLANCE. THIS ERROR | 80 |
| 10 | PREVENTED THE PRESSURE SWITCH FROM SENSING TRUE REACTOR (CONT.) | 80 |

| | | | | | | | | | |
|-----------------------------------|--|-------------------------|--|--------------------------|--|---------------------------|--|-----------------------------|--|
| FACILITY STATUS 11 D | | % POWER 000 | | OTHER STATUS NA | | METHOD OF DISCOVERY A | | DISCOVERY DESCRIPTION NA | |
| FORM OF ACTIVITY RELEASED 12 Z | | CONTENT OF RELEASE Z | | AMOUNT OF ACTIVITY NA | | LOCATION OF RELEASE NA | | | |

PERSONNEL EXPOSURES

| | | | | |
|----|---------------|-----------|-------------------|----|
| 13 | NUMBER 000 | TYPE Z | DESCRIPTION NA | 80 |
|----|---------------|-----------|-------------------|----|

PERSONNEL INJURIES

| | | | |
|----|---------------|-------------------|----|
| 14 | NUMBER 000 | DESCRIPTION NA | 80 |
|----|---------------|-------------------|----|

OFFSITE CONSEQUENCES

| | | |
|----|----|----|
| 15 | NA | 80 |
|----|----|----|

LOSS OR DAMAGE TO FACILITY

| | | | |
|----|-----------|-------------------|----|
| 16 | TYPE Z | DESCRIPTION NA | 80 |
|----|-----------|-------------------|----|

PUBLICITY

| | | |
|----|----|----|
| 17 | NA | 80 |
|----|----|----|

ADDITIONAL FACTORS

| | | |
|----|----|----|
| 18 | NA | 80 |
|----|----|----|

| | | |
|----|--|----|
| 19 | | 80 |
|----|--|----|

8303310552 760121
PDR ADOCK 05000237
S PDR

NAME: G. ROMBA

PHONE: Ext. 265

CAUSE DESCRIPTION (Continued)

pressure. The closed valve trapped full operating pressure in the line to the pressure switch. Since the switch did not activate when the actual pressure went below 600 psig, the bypass circuitry did not operate, and a full scram occurred. (Both 600 psig switches are required to complete the bypass circuit).

The reactor protection system functioned as designed, scrambling the reactor upon sensing pressure greater than 600 psig and a loss of condenser vacuum (heat sink).

The instrument maintenance supervisor emphasized to the individual involved the importance of returning a system to normal service after completion of a surveillance.

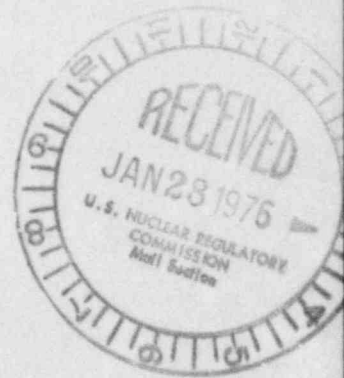


Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

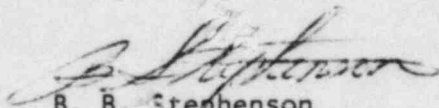
BBS Ltr. #53-76

January 21, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence number 50-237/1976-2.
This report is being submitted to your office in accordance with the
Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.


B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:smp

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC