

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 01 I L D R S 2 14 15 00 - 00 00 00 - 00 25 26 4 1 1 1 1 1 30 31 0 3 32

CATEGORY: 01 CON'T 57 58 REPORT TYPE: L 59 REPORT SOURCE: L 60 DOCKET NUMBER: 0 5 0 - 0 2 3 7 61 68 EVENT DATE: 0 3 2 3 7 6 69 74 REPORT DATE: 0 4 N 9 7 6 75 80

EVENT DESCRIPTION

02 7 8 9 DURING LOCAL LEAK-RATE TESTING, VALVES 2-1601-32A AND -32E
03 7 8 9 EXCEEDED THE INDIVIDUAL PENETRATION LEAK-RATE LIMIT OF 29.381
04 7 8 9 SCFH. ADDITIONALLY, THE TOTAL THROUGH-LEAKAGE FOR DOUBLE-GASKETED
05 7 8 9 SEALS EXCEEDED 10% LTO. THE LEAKAGE PATH WAS THROUGH THE
06 7 8 9 VALVE PACKING ALONG THE DISC SHAFTS. THIS EVENT IS A REPETITIVE
07 7 8 9 (SEE ATTACHED SHEET) 80

SYSTEM CODE: S A 9 10 CAUSE CODE: E 11 COMPONENT CODE: V A L V E X 12 17 PRIME COMPONENT SUPPLY: N 43 COMPONENT MANUFACTURER: A 5 8 5 44 47 VIOLATION: Y 48

CAUSE DESCRIPTION

08 7 8 9 THE LEAKAGE WAS CAUSED BY DETERIORATION OF THE WATERLESS GREASE
09 7 8 9 USED WITH DOUBLE TEFLON PACKINGS TO PROVIDE SHAFT SEALING.
10 7 8 9 CORRECTIVE ACTION WAS TO LUBRICATE ALL SHAFT SEALS. VALVES
11 7 8 9 (SEE ATTACHED SHEET) 80

FACILITY STATUS: H 9 % POWER: 0 0 0 10 12 13 OTHER STATUS: NA 44 45 46 METHOD OF DISCOVERY: B 44 45 46 DISCOVERY DESCRIPTION: NA 80

FORM OF ACTIVITY RELEASED: Z 9 CONTENT OF RELEASE: Z 10 11 AMOUNT OF ACTIVITY: NA 44 45 46 LOCATION OF RELEASE: NA 80

PERSONNEL EXPOSURES

13 7 8 9 NUMBER: 0 0 0 11 TYPE: Z 12 DESCRIPTION: NA 13 80

PERSONNEL INJURIES

14 7 8 9 NUMBER: 0 0 0 11 12 DESCRIPTION: NA 80

OFFSITE CONSEQUENCES

15 7 8 9 NA 80

LOSS OR DAMAGE TO FACILITY

16 7 8 9 TYPE: Z 10 DESCRIPTION: NA 80

PUBLICITY

17 7 8 9 NA 80

8303310254 760419
PDR ADOCK 05000237
S PDR

ADDITIONAL FACTORS

18 7 8 9 NA 80

19 7 8 9 NA 80

NAME: R. W. COEN

PHONE: EXT. 443

EVENT DESCRIPTION (continued)

occurrence which was first reported during the 1974-1975 Unit-2 refueling outage (see report nos. 50-237/1974-62 and -64). Since ten of the twelve valves were found to be within the individual penetration leak-rate limit, and since the unit had been in the shutdown condition for over a week, this event was considered to have little safety significance. (50-237/1976-14).

CAUSE DESCRIPTION (continued)

1601-32A and -32E were then retested, exhibiting a calculated individual leakage of 0.090 SCFH and 0.0 SCFH, respectively. This leakage reduction was reflected in the total through-leakage for double-gasketed seals, which was calculated at 2.381 SCFH, or well below 10% Lto.

Both valves are 18-inch tilting-disc swing check valves, model 60 LCB, spl. manufactured by Atwood and Morrill Co.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Lanthorn

BBS Ltr. #322-76

April 19, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence number 50-237/1976-14.
This report is being submitted to your office in accordance with the
Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:smp

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

4-82