

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 8 6				PAGE (3) 1 OF 0 2		
TITLE (4) Unit Trip																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 6	1 6	8 4	8 4	0 0 9	0 0	0 7	1 3	8 4					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
N		20.402(b)				20.405(e)				<input checked="" type="checkbox"/> 80.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				80.36(e)(1)				80.73(a)(2)(v)				73.71(e)		
1 0 0		20.405(a)(1)(ii)				80.36(e)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME John J. Anderson										TELEPHONE NUMBER AREA CODE 9 1 4 7 3 9 - 8 2 0 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
B	S J	F C V	C 1 6 1 3 1 5	Y												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 16, 1984, a reactor trip and subsequent unit trip were initiated automatically on a low level in No. 33 steam generator. Reactor power was 100 percent at the time of the trip. Investigation determined that the main feedwater regulating valve for No. 33 steam generator had inadvertently closed due to the failure of the valve operator's yoke. The valve operator was replaced, and the unit was returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 8 4 - 0 0 9 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At 0221 hours on June 16, 1984, a reactor trip and subsequent unit trip were initiated automatically on a low level in No. 33 steam generator. Reactor power was 100 percent at the time of the trip.

Investigation determined that the low level in No. 33 steam generator was caused by the inadvertent closure of FCV-437 (Copes-Vulcan, Serial No. 6810.67430.2.3), the main feedwater regulating valve for loop No. 33. The lower portion of the valve operator's yoke was found severed on one side. Analysis of the failure area determined that a crack had been present prior to the failure. The crack propagated through the yoke, aided by the force of the operator's spring. When the yoke failed the cap screws attaching the yoke assembly to the diaphragm also failed. The spring force was then released, closing the valve, and causing the reactor to trip on a low steam generator level.

A metallurgical evaluation of the yoke found the piece to be cast iron rather than cast steel, as was indicated on vendor drawings. The vendor has been notified of this problem. New valve operators with steel yokes have been ordered to preclude future failures. It is expected that these will be installed during the upcoming mid-cycle outage planned for October, 1984.

The operator for FCV-437 was replaced and the valve was returned to service. All equipment associated with the unit trip performed its designated function. The unit was synchronized to the bus at 2046 hours on June 16, 1984.

No similar event has been reported in an IER to date. This event is reportable under 10CFR50.73(a)(2)(iv), which became effective January 1, 1984.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200



July 13, 1984
IP-FWG-2846

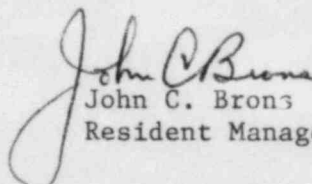
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 84-009-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,


John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
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