

LICENSEE EVENT REPORT

PREVIOUS REPORT DATE: 4/23

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

<p>LICENSEE NAME</p> <p>01 I L D R S 2</p> <p style="text-align: center;">7 8 9 14 15</p>	<p>LICENSE NUMBER</p> <p>0 0 - 0 0 0 0 0 - 0 0</p> <p style="text-align: center;">25 26 30 31 32</p>	<p>LICENSE TYPE</p> <p>4 1 1 1 1</p>	<p>EVENT TYPE</p> <p>0 3</p>
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<p>01</p> <p style="text-align: center;">7 8</p>	<p>CATEGORY</p> <p>CONT</p> <p style="text-align: center;">57 58</p>	<p>REPORT TYPE</p> <p>L</p> <p style="text-align: center;">59</p>	<p>REPORT SOURCE</p> <p>L</p> <p style="text-align: center;">60</p>	<p>DOCKET NUMBER</p> <p>0 5 0 - 0 2 3 7</p> <p style="text-align: center;">61 68</p>	<p>EVENT DATE</p> <p>0 3 2 4 7 6</p> <p style="text-align: center;">69 74</p>	<p>REPORT DATE</p> <p>0 6 2 5 7 6</p> <p style="text-align: center;">75 80</p>
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EVENT DESCRIPTION

02 DURING LOCAL LEAK-RATE TESTING, SRLC ISOLATION CHECK VALVE 2-1101-16 EXCEEDED

03 THE 29.321 SCFH LIMIT WITH A LEAKAGE OF 43.679 SCFH. VALVE 1101-15, WHICH IS IN

04 SERIES WITH 1101-16, EXHIBITED AN ACCEPTABLE LEAK RATE OF 21.314 SCFH. TOTAL THROUGH-

05 LEAKAGE, THEREFORE, WAS WITHIN THE TECH SPEC LIMITS FOR THE PENETRATION. THIS

06 IS NOT A REPETITIVE OCCURRENCE. (50-237/1976-17)

<p>07</p> <p style="text-align: center;">7 8 9 10</p>	<p>SYSTEM CODE</p> <p>S H</p>	<p>CAUSE CODE</p> <p>F</p> <p style="text-align: center;">11</p>	<p>COMPONENT CODE</p> <p>V A L V E X</p> <p style="text-align: center;">12 17</p>	<p>PRIME COMPONENT SUPPLIER</p> <p>N</p> <p style="text-align: center;">43</p>	<p>COMPONENT MANUFACTURER</p> <p>C 6 6 S</p> <p style="text-align: center;">44 47</p>	<p>VIOLATION</p> <p>Y</p> <p style="text-align: center;">48</p>
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CAUSE DESCRIPTION

08 AT THE TIME OF THE INITIAL REPORT, MAINTENANCE COULD NOT BE PERFORMED ON THE VALVE DUE

09 TO THE NEED FOR SBLC SYSTEM OPERABILITY. A SUBSEQUENT LLRT WAS PERFORMED ON THE VALVE

10 AFTER CLEAN DEMIN WATER HAD BEEN RUN THROUGH THE LINE FOR THE REQUIRED REFUELING SURVEIL-

(CONTINUED BELOW)

<p>11</p> <p style="text-align: center;">7 8 9</p>	<p>FACILITY STATUS</p> <p>H</p>	<p>% POWER</p> <p>0 0 0</p> <p style="text-align: center;">10 12 13</p>	<p>OTHER STATUS</p> <p>NA</p> <p style="text-align: center;">44</p>	<p>METHOD OF DISCOVERY</p> <p>B</p> <p style="text-align: center;">45</p>	<p>DISCOVERY DESCRIPTION</p> <p>NA</p> <p style="text-align: center;">46 80</p>
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<p>12</p> <p style="text-align: center;">7 8 9</p>	<p>FORM OF ACTIVITY RELEASED</p> <p>Z</p>	<p>CONTENT OF RELEASE</p> <p>Z</p> <p style="text-align: center;">10 11</p>	<p>AMOUNT OF ACTIVITY</p> <p>NA</p> <p style="text-align: center;">44 45</p>	<p>LOCATION OF RELEASE</p> <p>NA</p> <p style="text-align: center;">80</p>
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PERSONNEL EXPOSURES

<p>13</p> <p style="text-align: center;">7 8 9</p>	<p>NUMBER</p> <p>0 0 0</p> <p style="text-align: center;">11 12 13</p>	<p>TYPE</p> <p>Z</p>	<p>DESCRIPTION</p> <p>NA</p> <p style="text-align: center;">80</p>
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PERSONNEL INJURIES

<p>14</p> <p style="text-align: center;">7 8 9</p>	<p>NUMBER</p> <p>0 0 0</p> <p style="text-align: center;">11 12</p>	<p>DESCRIPTION</p> <p>NA</p> <p style="text-align: center;">80</p>
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OFFSITE CONSEQUENCES

15 NA

7 8 9 80

LOSS OR DAMAGE TO FACILITY

<p>16</p> <p style="text-align: center;">7 8 9</p>	<p>TYPE</p> <p>Z</p> <p style="text-align: center;">10</p>	<p>DESCRIPTION</p> <p>NA</p> <p style="text-align: center;">80</p>
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PUBLICITY

<p>17</p> <p style="text-align: center;">7 8 9</p>	<p>NA</p>	<p>8303310164 760625 PDR ADOCK 05000237 S PDR</p> <p style="text-align: center;">80</p>
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ADDITIONAL FACTORS

18 LANCE ACTUATION TEST. THIS LLRT YIELDED A SATISFACTORY LEAK RATE OF 2.645 SCFH. THE INITIAL

7 8 9 80

19 FAILURE, THEREFORE, WAS APPARENTLY CAUSED BY SOME SLIGHT CRUD BUILD-UP ON THE VALVE SEAT.

7 8 9 80

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Telephone 815/942-2920

6-25-76

BBS Ltr. #480-76

June 25, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find an update report to Reportable Occurrence number 50-237/1976-17. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

Label for
B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

COPY SENT REGION III

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