

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 1		PAGE (3) 1 OF 0 1			
TITLE (4) INADVERTENT DE-ENERGIZATION OF EMERGENCY CHILLER															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)				
0 6	1 4	8 4	8 4	0 2 9	0 0	0 7	1 2	8 4	SONGS UNIT 3		0 5 0 0 0 3 6 2				
OPERATING MODE (9) 1			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)												
POWER LEVEL (10) 1 0 0		20.402(b)				20.405(c)				X 50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)				50.36(c)(1)						73.71(c)			
		20.405(a)(1)(ii)				X 50.36(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER 7 1 1 4 4 9 2 - 1 7 7 0 0					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC					
SUPPLEMENTAL REPORT EXPECTED (14)											EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)					X NO										

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 14, 1984, at 0344, with Unit 2 in Mode 1 at 100% power and Unit 3 in Mode 5, breaker 3B0417 (EIIS Component Identifier BKR) was inadvertently tripped de-energizing load center BQ, initiating the Toxic Gas Isolation System (EIIS System Identifier JE) and rendering Emergency Chiller E-336 (EIIS Component Identifier CHU) inoperable. Loss of an Emergency Chiller renders equipment inoperable on both units in rooms where chilled water is provided to cool ambient air. Thus, the loss of E-336 renders two inverters inoperable in each unit, and since the associated Action Statement of Limiting Condition for Operation (LCO) 3.8.3.1 for Modes 1 through 4 addresses only the loss of one inverter, LCO 3.0.3 was invoked. Load center BQ was immediately re-energized from breaker 2B0417 restoring E-336 to service at 0348, and LCO 3.0.3 was exited.

The cause of this incident was personnel error in tripping the wrong breaker during transfer of charging pump power supply for Engineered Safety Feature Actuation System subgroup relay testing. The operator was counselled on the significance of the incident and the necessity for following procedures. This incident is an isolated occurrence and no further corrective action is planned.

There are no reasonable or credible circumstances under which this event would have been more severe.

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PDR ADOCK 05000361
S PDR

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

July 12, 1984

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-029
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.36(c)(2), 50.73(a)(2)(i)(B), and 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving Limiting Condition for Operation 3.8.3.1. Since this occurrence involved a shared system between Units 2 and 3, a single LER for Unit 2 is enclosed per NUREG-1022. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER 84-029

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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