

CONTROL BLOCK: 1										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																			
01	C	A	S	O	S	3	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5			
LICENSEE CODE					LICENSE NUMBER										LICENSE TYPE					CAT 59									
CONT																													
01	L	6	0	5	0	0	0	3	6	2	7	0	5	0	9	8	3	8	0	7	d	d	8	4	9				
REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE												
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10																													
02	A contractor Quality Assurance (QA) inspection identified a partial disengagement																												
03	of the rod end bushing on the Mechanical Shock arrestor (snubber) installed on																												
04	pipe support P/S S3-CS-027-H-026. As a result, Bechtel QC initiated a																												
05	Non-Conformance Report (NCR) 3-336 on May 4, 1983, which was received, validated																												
06	and issued by SCE QA on May 9, 1983 (See Attachment).																												
07																													
08																													
09	S	B	11	B	12	B	13	S	U	P	O	R	T	14	D	15	Z	16											
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE					COMP. SUBCODE		VALVE SUBCODE																
17	8	3				0	1	3			0	3		X			1												
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.																			
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER 26													
B		Z		Z		Z		0		0		0		Y		N		Z		P									
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27																													
10	The snubber was inoperable because the bearing assembly was not properly secured																												
11	(by staking) as required by the code. The snubber was reworked and declared																												
12	operable at 0441 on 5/13/83. An engineering evaluation in accordance with																												
13	Surveillance Requirement 4.7.6.g determined that the SDCS was not adversely																												
14	affected by the snubber's inoperability. (See Attachment)																												
15	B	28	0	0	0	29	NA	30	C	31	Non-Routine Inspection 32																		
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION																					
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY		35		NA		NA		LOCATION OF RELEASE 36																			
16	Z	33	Z	34	NA		NA																						
PERSONNEL EXPOSURES NUMBER		TYPE		DESCRIPTION 39																									
17	0	0	0	37	Z	38	NA																						
PERSONNEL INJURIES NUMBER		DESCRIPTION 41																											
18	0	0	0	40	NA																								
LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION 43																											
19	Z	42	NA																										
PUBLICATION ISSUED DESCRIPTION 45		NA																											
20	N	44	NA																										
NAME OF PREPARER J. G. HAYNES																													
PHONE (714) 492-7700																													

ATTACHMENT TO LER 83-013, REVISION 1

SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION

UNIT NO. 3, DOCKET NO. 50-362

SUPPLEMENTAL INFORMATION FOR EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

With the Unit in Mode 5, this snubber on the common line for SDCS Train A and B was declared inoperable on May 9, 1983, at 1100. The Action Statement associated with LCO 3.7.6 requires that the snubber be returned to operable status within 72 hours. As the snubber was not returned to operable status within 72 hours, the SDCS was declared inoperable and LCO 3.4.1.4.1 Action Statement 'a' for the SDCS was invoked on May 12, 1983, at 1100. Public health and safety were not affected.

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

Inspection of the disengaged bearing has shown that loads imposed on the snubber have not damaged the bearing assembly. Therefore, the ability of the snubber to perform as designed has not been limited by the partial disengagement of the bearing. Consequently, no adverse effects to components attached to this snubber have occurred. An in-service snubber visual inspection within ten months of commercial operation required by Technical Specifications 4.7.6.b was performed in accordance with SO23-I-2.29, Rev. 5, on January 23 through 31, 1984. All Unit 3 and common snubbers have been determined to be operable, except for one snubber (S3-BM-057-H-003) in the Boric Acid Make-up line that was discovered to have a missing lower clevis pin, as documented in NCR 3-724. An engineering evaluation in accordance with Surveillance Requirement 4.7.6.g determined that the inoperability of the snubber had no effect on the associated piping or components. As corrective action, the clevis pin was installed and the snubber rendered operable.

Retraining of Bechtel and Southern California Edison Quality Assurance personnel to emphasize the importance of expediting the processing of Nonconformance Reports was completed on June 10, 1983.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

July 6, 1984

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SCE

REGION V IRE

TELEPHONE
(714) 492-7700

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362
Licensee Event Report No. 83-013, Revision 1
San Onofre Nuclear Generating Station, Unit 3

Reference: Letter, H. B. Ray (SCE) to J. B. Martin (USNRC),
dated June 8, 1983, Licensee Event Report No. 83-013

The referenced letter provided the required 30-day Licensee Event Report (LER) for an occurrence involving Limiting Conditions for Operation (LCO's) 3.7.6 and 3.4.1.4.1 associated with an inoperable snubber on the Shutdown Cooling System (SDCS). In that LER, we reported that a revised LER would be submitted to provide the results of snubber surveillance required by Technical Specifications 4.7.6.b and the corrective actions taken. Enclosed is LER 83-013, Revision 1.

If you require any additional information, please so advise.

Sincerely,

J. G. Haynes

Enclosure: LER No. 83-013, Revision 1

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission
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Institute of Nuclear Power Operations (INPO)

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