

UPDATE REPORT - PREVIOUS REPORT DATE NOVEMBER 18, 1983

NRC FORM 366
(12-81)
10 CFR 50U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 C A S O S 2 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

01 REPORT SOURCE L 6 0 5 0 0 0 3 6 1 7 1 0 1 9 8 3 8 0 7 0 6 8 4 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 10/19/83, at 0815, with Unit 3 in Mode 1, Unit 3 Train A Containment

03 Emergency Cooling Unit 3E-399 failed to meet the acceptance criteria of

04 Technical Specification 4.6.2.3.a.2, and was declared inoperable and LCO 3.6.2.3,

05 Action Statement 'a', was entered. On 10/20/83, at 1000, with Unit 2 in Mode 1,

06 Unit 2E-399 failed in the same manner, and Unit 2 entered LCO 3.6.2.3, Action

07 Statement 'a'. There was no effect on public health or safety because of these

08 events.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
S B 11 B 12 C 13 V A L V E X 14 E 15 D 16
17 LER/RO REPORT NUMBER 8 3 13 7 0 3 X 1
21 22 23 24 25 26 27 28 29 30 31 32
ACTION FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
E 18 Z 19 Z 20 Z 21 0 0 0 0 0 Y 23 Y 24 A 25 L 2 0 0 0
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The events occurred when 3HV-6371 and 2HV-6371, respectively, failed to open

11 completely to allow a flow rate of 2000 gpm. In both cases, the torque settings

12 on the valves were increased, and the systems were declared operable per

13 S023-3-3.30 at 0140 on 10/21/83 and at 1645 on 10/21/83, for Units 2 and 3

14 respectively. (See attachment.)

15 FACILITY STATUS 1 0 0 28 29 OTHER STATUS 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Test
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

16 ACTIVITY CONTENT RELEASED OR RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

18 PERSONNEL INJURIES NUMBER DESCRIPTION 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

20 PUBLICITY ISSUED DESCRIPTION 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER J. G. HAYNES

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PDR ADOCK 05000361
S PDRIE 29
111

ATTACHMENT TO LER 83-137, REVISION 1

SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION

UNIT NO. 2, DOCKET NO. 50-361

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

Investigation revealed that the failures of the valves to fully open were due to inadequate adjustment of torque switch settings. As corrective action, the training program has been modified to reflect the proper adjustment of torque switch setting for Component Cooling Water (CCW) containment isolation valves. Training begins July 9, 1984. In addition, we have increased the motor size for both Units 2 and 3 CCW containment isolation valves to improve the design margin and ensure proper operation of the valves.

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Southern California Edison Company

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SCE

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

REGION VISE

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

July 6, 1984

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket Nos. 50-361 and 50-362
Licensee Event Report No. 83-137, Revision 1
(Docket No. 50-361)
San Onofre Nuclear Generating Station, Units 2 and 3

Reference: Letter, J. G. Haynes (SCE) to J. B. Martin (USNRC),
dated November 18, 1983, Licensee Event Report No. 83-137
(Docket No. 50-361)

The referenced letter provided the required 30-day Licensee Event Report (LER) pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating Licenses NPF-10 and NPF-15 for San Onofre Units 2 and 3, respectively, addressing two occurrences involving Limiting Condition for Operation (LCO) 3.6.2.3 associated with Containment Cooling Systems. Since the two occurrences involved the same components, system, cause and method of discovery, these events have been combined into a single report per NUREG-1022. Enclosed LER 83-137, Revision 1, provides the results of the investigation into the cause and corrective actions taken.

1E-24

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If there are any questions regarding these events, please so advise.

Sincerely,

W. H. Haynes

Enclosure: LER 83-137, Revision 1

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission
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Institute of Nuclear Power Operations (INPO)