

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD WEST
SYRACUSE, N. Y. 13202

April 9, 1974

Mr. Donald J. Skovholt
Assistant Director for Reactor Operations
Division of Reactor Licensing
United States Atomic Energy Commission
Washington, D. C. 20545



Re: Provisional Operating License: DPR-17
Docket No.: 50-220
Abnormal Occurrence - 50-220-74-3

Dear Mr. Skovholt:

On April 2, 1974, in accordance with Technical Specification 4.3.3 e(4), the four (4) Main Steam Isolation Valves were leak tested. The outside MSIV's in each line successfully passed the leak rate test, however the inside valves in both lines failed to meet the required limit of 12.9 SCFH. Notification was made to RO:I on April 2, 1974 in accordance with Technical Specification 1.13e.

The Main Steam Flow from the reactor to the turbine is through two independent 24" lines. Each line contains two isolation valves, one inside the containment and one immediately outside the containment. The two failed valves, one in each line, were the valves inside the containment.


Under containment design basis accident conditions, both main steam lines would be essentially leak tight with low leakage through the outside valves and no hazard would be presented to the plant or the general public.

The primary corrective measure taken will depend upon the cause of leakage. It appears from preliminary results that one valve (#111) experienced a drift in the position switch on the valve controller to cause the valve not to fully close. The other valve #121, appears to require machine lapping of the valve seat. Such repairs have been made in the past and this is not the first of a kind failure.¹

The tabular results for each valve is indicated below:

#111 Inside	1400 SCFH
#112 Outside	7.85 SCFH
#121 Inside	120 SCFH
#122 Outside	0.24 SCFH

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PDR ADOCK 05000220
S PDR

COPY SENT REGION 

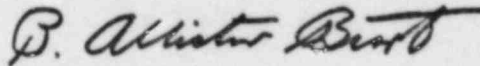
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In the interest of maintaining an efficient refueling outage schedule, the repair and retesting of the inside valves has not been completed to this date, however, the results of the repair and retest will appear in the Semi-Annual report published in July, 1974.

Very truly yours,



P. Allister Burt
General Superintendent
Nuclear Generation

TJD:cm

¹Ltr. P.A. Burt - Skovholt - June 6, 1973.