


Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

MEMBER OF THE
General  Public Utilities Corporation

April 18, 1975



Mr. A. Giambusso
Director, Division of Reactor Licensing
Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D. C. 20555


Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/75-11

The purpose of this letter is to forward to you the attached abnormal occurrence report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Enclosed are forty copies of this submittal.

Very truly yours,


Donald A. Ross, Manager
Generating Stations-Nuclear

cs
Enclosures

cc: Mr. J. P. O'Reilly, Director
Office of Inspection and Enforcement, Region 1

*50-219
inclosed*

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General



Public Utilities Corporation

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/75-11

Report Date

April 18, 1975

Occurrence Date

April 10, 1975

Identification of Occurrence

Violation of the Technical Specifications, paragraph 4.5.F.1.d, leakage of the main line drain and bypass line in excess of the 5% L_{to} allowable for any one penetration or isolation valve. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15.E.

Conditions Prior to Occurrence

The reactor was shut down for refueling. The reactor coolant temperature was less than 212°F and the reactor mode switch in refuel.

Description of Occurrence

On April 10, 1975, at 1100 hours, a local leak rate test was conducted on the main steam line drain and bypass line (V-1-106, 107, 110, and 111), and the leakage rate was calculated to be 38.3 scfh when corrected to 20 psig pressure. While pressurizing the line, leakage was detected past the packing of V-1-110.

Apparent Cause of Occurrence

Failure of the packing on valve V-1-110 is the cause of this occurrence.

Analysis of Occurrence

This event cannot be fully analyzed until repairs are made to V-1-110. At that time, the line will be tested again to determine the leakage. If required, further testing will be conducted to determine the leakage of the inside valves

(V-1-106 and 107) and the outside valves (V-1-110 and 111), separately, to determine if a failure of redundant components had occurred.

Corrective Action

Valve V-1-110 will be repacked and then retested to assure its leak rate is less than allowable. The Plant Operations Review Committee concluded that repacking the valve is sufficient repair action for this type of failure.