

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket No. 50-282
50-306

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NOS. DPR-42 & DPR-60

License Amendment Request Dated July 9, 1984

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This letter contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By David Musolf
David Musolf
Manager - Nuclear Support Services

On this 9th day of July, 1984 before me a notary public in and for said County, personally appeared David Musolf, Manager - Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

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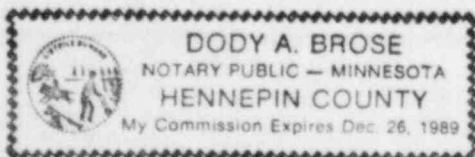


Exhibit A

Prairie Island Nuclear Generating Plant

License Amendment Request Dated July 9, 1984
Evaluation of Proposed Changes to the
Technical Specifications Appendix A of
Operating License DPR-42 and DPR-60

Pursuant to 10 CFR Part 50, Sections 50.59 and 50.90 the holders of Operating Licenses DPR-42 and DPR-60 hereby propose the following change to Appendix A, Technical Specifications:

STEAM GENERATOR TUBE SURVEILLANCE

Proposed Change

Add the following exception to paragraph TS.4.12.D.2:

"Except for three tubes identified in a letter dated June 18, 1984 from NSP to Director of NRR during Unit 1, Cycle 9."

Reason For Change

Three eddy current indications were recently discovered through re-evaluation of eddy current test data to exceed the Technical Specification plugging criteria. This Technical Specification change is required because the NRC Staff has determined that the Technical Specification requiring plugging of these tubes is applicable during plant operations.

Significant Hazards Evaluation

The eddy current indications suggest that three tubes are potentially degraded. These indications are located in the tube sheet crevice area. Also, the eddy current signals suggest short indications less than 0.5 inches. Due to the location in the tubesheet crevice area, any leakage that does occur would be restricted to less than 10 gpm if all three tubes would fail under accident conditions. The length of the tubesheet above the elevation of the indications creates a tight annulus around the tube that acts as a leak-limiting orifice. This constraining annulus will also act against tube burst and double ended rupture by restricting the crack opening and the tube separation if the tube should continue to degrade. Further information is provided in our June 18, 1984 letter.

This change will only be effective during the remainder of Unit 1, Cycle 9 operation (approximately 6 months). The steam jet air ejector monitor and secondary side chemistry samples will identify any degradation of the tubes. These three identified tubes will be plugged at the next refueling outage.

For these reasons, operation of Unit 1 of the Prairie Island Nuclear Generating Plant in accordance with this proposed change will not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) involve a significant reduction in a margin of safety.