

Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

General  Public Utilities CorporationOYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731Abnormal Occurrence
Report No. 50-219/75-27Report Date

October 17, 1975

Occurrence Date

October 8, 1975

Identification of Occurrence

Violation of Technical Specifications, paragraph 3.1.1.D.3, Low Reactor Pressure Core Spray Valve Permissive Pressure Switches RE17B and D were found to trip at pressures less than the minimum required value of 285 psig. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15.B.

Conditions Prior to Occurrence

The plant was at steady state power with the following major parameters:

Power:	Core, 582.6 MWt
	Electric, 155 MWe
Flow:	Recirculation, 6.0×10^4
	Feedwater, 1.77×10^6 lb/hr
Stack Gas:	3090 μ ci/sec

Description of Occurrence

On Wednesday, October 8, 1975, at approximately 1130, while performing quarterly surveillance testing on the four (4) Low Reactor Pressure Core Spray Valve Permissive Pressure Switches, it was discovered that RE17B and D tripped at 275 psig and 277 psig, respectively. These values are less than the Technical Specification limit of 285 psig. Pressure switches RE17B and D were immediately recalibrated.

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The "As Found" and "As Left" switch settings were:

	<u>"As Found" Settings</u>	<u>"As Left" Settings</u>
RE17A	285 psig	285 psig
RE17B	275 psig	287 psig
RE17C	285 psig	285 psig
RE17D	277 psig	289 psig

Apparent Cause of Occurrence:

The cause of this occurrence is switch repeatability.

Analysis of Occurrence

The core spray system parallel isolation valves open when a low-low reactor water level and/or high drywell pressure condition exists in addition to a low reactor pressure condition (285 psig). The four (4) low reactor pressure core spray valve permissive pressure switches sense the low reactor pressure condition and provide signals to the valve opening logic. Two (2) of these switches (RE17A and B) are associated with Core Spray System 1 and the other two (2) switches (RE17C and D) are associated with Core Spray System 2. A trip of one switch in each core spray system is required to effect parallel isolation valve opening in that system. A review of the "as found" switch settings indicates that parallel isolation valves in both core spray systems would have opened at reactor pressure of 285 psig had a reactor low-low water level and/or high drywell pressure condition existed concurrently. The safety significance of this event is considered to be the loss of switch redundancy.

Corrective Action

Immediate corrective action involved the recalibration of pressure switches RE17B and D. There are continuing efforts to resolve the incompatibilities between the Technical Specification setpoint limits and the sensor performance limits. It is felt that the conservative design margins associated with the derivation of the plant safety limits will permit a change in the Technical Specifications to be made which will take into account the expected sensor performance variations. This will eliminate instances of abnormal occurrence reports caused by the normal variation in a sensor setpoint within the design margins of the plant safety limits.

Failure Data

Manufacturer:	Barksdale
Type:	Pressure Actuated Switch
Range:	50-1200 psig
Switch No.:	B2T-A12SS (RE17B) B2T-A12SS (RE17D)


Previous abnormal occurrence reports involving these switches are:

1. Abnormal Occurrence Report No. 50-219/74-9
2. Abnormal Occurrence Report No. 50-219/74-10
3. Abnormal Occurrence Report No. 50-219/74-12
4. Abnormal Occurrence Report No. 50-219/74-22
5. Abnormal Occurrence Report No. 50-219/74-35
6. Abnormal Occurrence Report No. 50-219/74-37
7. Abnormal Occurrence Report No. 50-219/74-41
8. Abnormal Occurrence Report No. 50-219/74-42
9. Abnormal Occurrence Report No. 50-219/74-43
10. Abnormal Occurrence Report No. 50-219/74-51
11. Abnormal Occurrence Report No. 50-219/75-12

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October 17, 1975



Mr. R. Boyd, Acting Director
Division of Reactor Licensing
Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Boyd:

IE FILE COPY

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/75-27

The purpose of this letter is to forward to you the attached abnormal occurrence report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Very truly yours,

Donald A. Ross, Manager
Generating Stations-Nuclear

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Enclosures

cc: Mr. J. P. O'Reilly, Director
Office of Inspection and Enforcement, Region 1

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