

Initial Telephone  
Report Date: November 6, 1975

Date of  
Occurrence: November 6, 1975

Initial Written  
Report Date: November 7, 1975

Time of  
Occurrence: approx. 10:30 AM

Oyster Creek Nuclear Generating Station  
Forked River, New Jersey 07731

Abnormal Occurrence  
Report No. 50-219/75/30

IDENTIFICATION OF OCCURRENCE: Violation of the Technical Specifications, paragraph 3.1.1.D.3  
Low Reactor Pressure Core Spray Valve Permissive Pressure Switches  
RE 17 A and C was found to trip at pressures less than the minimum required value of  
265 psig.

This event is considered to be an abnormal occurrence as de-  
fined in the Technical Specifications, paragraph 1.15B.

CONDITIONS PRIOR  
TO OCCURRENCE:

<input checked="" type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Safeguarding Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

Power: Core 1565 MWt  
Electrical 532 MWe  
Flow: Recirculation  $15.2 \pm 10^3$  GPM  
Feedwater  $5.75 \pm 10^3$  M/hr.  
Stack Gas 9,000  $\mu$  Ci/Sec.

DESCRIPTION OF OCCURRENCE: On Thursday and Friday, November 6 and 7, 1975 while performing the monthly  
surveillance testing on the four (4) Low Reactor Pressure Core Spray Valve  
Permissive Pressure Switches, it was discovered that Switches RE 17 A and  
C tripped at 277 psig and 282 psig, respectively. These values are less than the Technical  
Specification limit of 285 psig. The Pressure Switches RE 17 A and C were immediately  
recalibrated. The "as found" and "as left" switch settings were:

	"As Found" Settings	"As Left" Settings
RE 17 A	277 psig	285 psig
RE 17 B	286 psig	286 psig
RE 17 C	282 psig	285 psig
RE 17 D	288 psig	288 psig

APPARENT CAUSE  
OF OCCURRENCE:

Design  
Manufacture  
Installation/  
Construction  
Operator

Procedure  
Unusual Service Condition  
Inc. Environmental  
Component Failure  
X Other (Specify)

The cause of this occurrence is switch repeatability.

ANALYSIS OF

DESCRIPTION: The Core Spray System parallel isolation valves open when a low-low reactor water level and/or high drywell pressure condition exists in addition to a low reactor pressure condition (285 psig). The four (4) low reactor pressure core spray valve permissive switches of the low reactor pressure condition and provide signals to the valve opening logic. Two (2) of these switches (RE 17A and B) are associated with Core Spray System I and the other two (RE 17C and D) are associated with Core Spray System II. A trip of one switch in a Core Spray System is required to effect parallel isolation valve opening in that system. A review of the "as found" switch settings indicate that parallel isolation valves in both Core Spray Systems would have opened at a reactor pressure of 285 psig for System I and 288 psig for System II had a reactor low-low water level and/or high drywell pressure condition exist concurrently. The safety significance of this event is considered to be the loss of switch redundancy.

CORRECTIVE

ACTION: Immediate corrective action involved the recalibration of pressure switches RE17A and B. There are continuing efforts to resolve the incompatibilities between the Technical Specification setpoint limits and the sensor performance limits. It is felt that the conservative design associated with the deprivation of the plant safety limits will permit a change in the Technical Specifications to be made which will take into account the expected sensor performance variability. This will eliminate instances of abnormal occurrence reports caused by the normal variation in sensor setpoint within the design margins of the plant safety limits.

ALLIANCE DATA: Manufacturer: Beckhoff  
Type: Pressure Switch  
Range: 50-1200 psig  
Switch No.: BTT-ALTES (RE17A)  
BTT-WT255 (RE17C)  
Abnormal Occurrence 50-219/75/12  
Abnormal Occurrence 50-219/75/27

Prepared by:

John L. Sullivan

Date: November 7, 1975

TO:

James P. O'Sullivan  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

FROM:

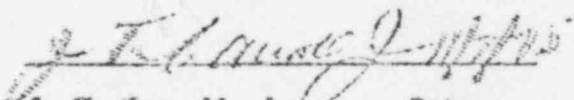
Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station Docket #50-21  
Forked River, New Jersey 08731

SUBJECT:

Abnormal Occurrence Report No. 50-219/75/ 30

The following is a preliminary report being  
submitted in compliance with the Technical  
Specifications, paragraph 6.6.2.

Preliminary Approval:

  
S. F. Gessell, Jr. Date

CC: Mr. A. Giambusso