

Initial Telephone
Report Date: January 3, 1976

Date of
Occurrence: January 2, 1976

Initial Written
Report Date: January 5, 1976

Time of
Occurrence: 2300

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Reportable Occurrence
Report No. 50-219/76-1/1P

IDENTIFICATION ~~VIOLATION OF THE Technical Specifications~~
OF OCCURRENCE: Failure of one of the High Drywell Pressure Switches associated with Core Spray Actuation to trip at a value equal to or less than 2 psig.

Reportable
This event is considered to be an ~~abnormal~~ occurrence as defined in the Technical Specifications, paragraph 1.15B.

CONDITIONS PRIOR
TO OCCURRENCE:

<u> </u> Steady State Power	<u> </u> Routine Shutdown
<u> </u> Hot Standby	<u> </u> Operation
<u> </u> Cold Shutdown	<u> </u> Load Changes During
<u>XXX</u> Refueling Shutdown	<u> </u> Routine Power Operation
<u> </u> Routine Startup	<u> </u> Other (Specify)
<u> </u> Operation	<u> </u>

The reactor mode switch was in the Refuel position with reactor coolant temperature approximately 100° F.

DESCRIPTION

OF OCCURRENCE: On Friday, January 2, 1976, at approximately 2300 while performing the monthly surveillance of the high drywell pressure initiation of Core Spray System II, the RV46B (High Drywell Pressure Switch) trip point was found to be 2.75 psig. Following replacement of the mercury switch bulb within the pressure switch, RV46B was retested and verified to trip at 2.0 psig. Tests on all the High Drywell Pressure Switches for the Core Spray Systems revealed the following data:

	Switch	Desired Setpoint	As Found	As Left
System I	RV46A	≈ 2 psig	2.00	2.00
	RV46C	≈ 2 psig	2.00	2.00
System II	RV46B	≈ 2 psig	2.75	2.00
	RV46D	≈ 2 psig	2.00	2.00

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/> Design	<input type="checkbox"/> Procedure
<input type="checkbox"/> Manufacture	<input type="checkbox"/> Unusual Service Condition
<input type="checkbox"/> Installation /	<input type="checkbox"/> Inc. Environmental
<input type="checkbox"/> Construction	<input checked="" type="checkbox"/> Component Failure
<input type="checkbox"/> Operator	<input type="checkbox"/> Other (Specify)

Component failure has been identified as the cause of this event.

ANALYSIS OF
OCCURRENCE:

Since Core Spray System II would be actuated by either RV46B or RV46D, this occurrence is considered a loss of redundancy in the Drywell High Pressure Switches associated with Core Spray System II. If a high drywell condition would have occurred, RV46D would have initiated Core Spray System II at 2.0 psig. In addition, the redundant Core Spray System I would have actuated at 2.0 psig by tripping of RV46A and RV46C. The only safety significance of this event is the loss of redundancy in the High Drywell Pressure Switches associated with System II.

CORRECTIVE
ACTION:

Corrective action involved replacing the mercury switch bulb in the pressure switch and successively actuating RV46B to demonstrate that proper response could be consistently obtained.

FAILURE DATA:

Manufacturer: Barton IPT
Type: Bellows - torque bar
Range: 0-10 psig
Model: 278

A previous abnormal occurrence report involving these switches was:

Abnormal Occurrence Report No. 50-219/75/20.

PREPARED BY: 

DATE: 1/5/76



TO:

James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

FROM:

Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-21
Forked River, New Jersey 08731

SUBJECT:

Reportable Occurrence Report No. 50-219/76-1/1P

The following is a preliminary report being
submitted in compliance with the Technical
Specifications, paragraph 6.6.2.

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Preliminary Approval:

J. T. Carroll, Jr. 11/26
J. T. Carroll, Jr. Date

CC: Mr. Roger Boyd

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