

Report Date: March 4, 1976

Initial Written Report Date: March 4, 1976

Date of Occurrence: March 3, 1976

Time of Occurrence: 1400

OYSTER CREEK NUCLEAR GENERATING STATION  
FORKED RIVER, NEW JERSEY 08731

Reportable Occurrence  
Report No. 50-219/76-5-1P

IDENTIFICATION  
OF OCCURRENCE:

~~Violation of the Technical Specifications, paragraph 1.1.5D~~  
Failure of Diesel Generator #2 to supply power to the emergency bus during a loss of power test.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 1.1.5D.

CONDITIONS PRIOR  
TO OCCURRENCE:

<input type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input checked="" type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The reactor mode switch was in the Refuel position with reactor coolant temperature approximately 141°F.

DESCRIPTION  
OF OCCURRENCE:

During the performance of the "Integrated Core Spray/Diesel Generator Loading" test procedure, which simulates a loss of power with a fault on the opposite bus, Diesel Generator #2 was required to assume the inced loads on the 1D bus in the fast start mode. Loss of power was simulated on the 1D by tripping the S1B breaker with 1D breaker tripping as it should. Diesel Generator #2 started in the fast start mode as it should but did not develop sufficient voltage to on to the 1D bus and assume the sequential loads. Operator action was to re-energize .B bus via the S1B breaker and the 1D bus via the 1D breaker to restore power to the emency bus.

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PDR ADOCK 05000219  
S PDR

PARENT CAUSE  
OCCURRENCE:-

Design  
Manufacture  
Installation/  
Construction  
Operator

Procedure  
XX Unusual Service Condition  
Inc. Environmental  
Component Failure  
Other (Specify)

Investigation of the problem revealed that excitation was not being applied to DG #2 generator the diesel generator batteries. The cause of the problem was found to be in the FFCO relay contacts which were found to be "chattering" during diesel generator operation. Because of the chattering the Field Flashing Contactor was not being energized for the purpose of electrically connecting the batteries to the excitation circuit for the generator. The FFCO relay is located in the diesel generator compartment and is subject to vibration from the diesel generator during operation possibly causing the "chatter" in the relay. Failure of this relay is in no way associated with the work that was performed in connection with the CCS Modification.

ANALYSIS OF OCCURRENCE: The failure of the FFCO relay contacts to energize the Field Flashing Contactor resulted in a loss of Standby power for bus 1D. Had a true loss of power occurred at this time, DG #2 would not have been able to supply power to the 1D bus. However, had coincident accident conditions required the operation of plant safety systems, the redundant safety system (those powered for the 1C bus) and DG #1 would have been available.

CORRECTIVE ACTION: To alleviate the problem of contact "chattering", the contact wipe was increased to overcome the vibration induced by the diesel generator. After adjustment of the relay contacts, a test was performed to simulate dead bus mode with subsequent proper functioning of the FFCO relay.

FAILURE DATA: Relay designation  
Manufacturer  
Style  
Type

FFCO  
Westinghouse  
1876094  
SV Voltage Relay  
120 Volts 60 Cycles.

Prepared by: J. E. Quintenz  
T. E. Quintenz

Date: March 4, 1976

TO:

James P. O'Reilly  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

*IDE*

FROM:

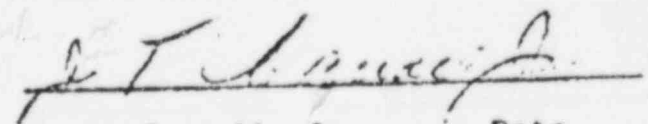
Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station Docket #50-1  
Forked River, New Jersey 08731

SUBJECT:

Reportable Occurrence Report No. 50-219/ 76<sup>5</sup>-1P

The following is a preliminary report being  
submitted in compliance with the Technical  
Specifications, paragraph 6.6.2.

Preliminary Approval:

  
J. T. Carroll, Jr. Date

CC: ROGER BOYD

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