

DUKE POWER COMPANY

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July 11, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: Catawba Nuclear Station
Docket No. 50-413

Dear Mr. Denton:

On March 19, 1984 representatives from Duke Power, Westinghouse, and the NRC met to discuss Confirmatory Item 12, Main Steamline Break using a revised Heat Transfer Model, which is discussed in Section 6.2.1.1 of the Catawba FSAR. Duke subsequently provided the staff with additional information concerning this subject in a letter dated March 28, 1984, and Westinghouse submitted for NRC review WCAP 8354 that documents Ice Condenser Containment analysis techniques. In that this issue involves the temperatures used for environmental qualification of equipment inside containment, and since the staff has not completed their review of this item, Duke submits the following analysis in accordance with 10 CFR 50.49 to support fuel loading and pre-critical testing activities.

Duke Power has reviewed this situation with respect to the pending request to load fuel and perform pre-critical testing for Catawba Unit 1. It was concluded that the consequences would be less severe than analyzed in FSAR Section 15.1.5 for the following reasons:

- 1) Prior to initial criticality, there would be no fission product inventory in the core.
- 2) The moderator temperature coefficient at beginning-of-life (BOL) is slightly positive. Therefore, the core would not go critical as a result of a MSLB accident.

Very truly yours,

H.B. Tucker
Hal B. Tucker

NAR/slb

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
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cc: NRC Resident Inspector
Catawba Nuclear Station

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