



**CENTERIOR
ENERGY**

PERRY NUCLEAR POWER PLANT

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U.S. Nuclear Regulatory Commission
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Perry Nuclear Power Plant
Docket No. 50-440
1991 Annual Report

Gentlemen:

Attached is the 1991 Annual Report for Perry Unit 1. This report is submitted in accordance with Technical Specifications 6.9.1.4 and 6.9.1.5 and fulfills ongoing commitments associated with Licensing Commitment 17 of USAR Appendix 1B.

Sincerely,

Michael D. Lyster

MDL:SML:ss

cc: NRC Project Manager
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THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

PERRY NUCLEAR POWER PLANT

JANUARY 1 - DECEMBER 31, 1991

ANNUAL REPORT

DOCKET NUMBER: 50-440

LICENSE NUMBER: NPF-58

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I. OCCUPATIONAL RADIATION EXPOSURE SUMMARY REPORT

Enclosed in this section is the 1991 Annual Occupational Exposure Summary Report required by Technical Specification 6.9.1.5.a of Appendix A to the Perry Unit 1 Operating License. The dose assignments to various duty functions provided herein were obtained from pocket dosimeter measurements recorded in Radiation Work Permit (RWP) documents. Thermoluminescent dosimeter (TLD) measurements for 1991 are provided under a separate report in accordance with 10 CFR 20.407. Work categories were grouped as indicated below:

STATION Perry Nuclear Power Plant Department, Perry Nuclear Support Department, Performance Engineering Section and Systems Engineering Section of the Perry Nuclear Engineering Department

UTILITY Perry Nuclear Assurance Department and all other Sections of the Perry Nuclear Engineering Department

OTHER Contractor, Consultant, Centerior, Davis-Besse, Eastern District, and other support personnel

1991 ANNUAL EXPOSURE REPORT
REGULATORY GUIDE 1.16 APPENDIX A SUMMARY

WORK CATEGORY	JOB CATEGORY	# OF PERSONS STATION	> 5100 MREM UTILITY	OTHER	STATION	MAX-REM UTILITY	OTHER
REACTOR OPER. & SURVEILLANCE	MAINTENANCE	18		4	0.247		0.000
	OPERATIONS	94		5	12.811		0.031
	HEALTH PHYSICS	40		6	9.863		3.255
	SUPERVISORY						
	ENGINEERING		12	1		0.300	0.070
ROUTINE MAINTENANCE	MAINTENANCE	98		72	4.438		2.689
	OPERATIONS	64		6	5.133		0.375
	HEALTH PHYSICS	33		16	0.613		0.372
	SUPERVISORY						
	ENGINEERING	1	12	2	0.415	0.104	0.010
INSERVICE INSPECTION	MAINTENANCE	9		17	0.075		1.090
	OPERATIONS			1			0.000
	HEALTH PHYSICS	13		4	0.285		0.245
	SUPERVISORY						
	ENGINEERING		3	1		0.618	0.395
SPECIAL MAINTENANCE	MAINTENANCE	100		87	32.834		36.924
	OPERATIONS	94		7	9.626		0.795
	HEALTH PHYSICS	39		16	15.257		4.135
	SUPERVISORY						
	ENGINEERING	1	14	2	0.015	4.246	0.310
WASTE PROCESSING	MAINTENANCE	74		22	1.450		3.225
	OPERATIONS	32		2	3.100		0.960
	HEALTH PHYSICS	33		15	4.245		0.925
	SUPERVISORY						
	ENGINEERING		2			0.075	
REFUELING	MAINTENANCE						
	OPERATIONS						
	HEALTH PHYSICS						
	SUPERVISORY						
	ENGINEERING						
TOTALS	MAINTENANCE	299		202	39.714		43.928
	OPERATIONS	284		21	30.670		2.161
	HEALTH PHYSICS	158		67	30.263		8.932
	SUPERVISORY						
	ENGINEERING	2	43	6	0.430	5.339	0.735
GRAND TOTALS		743	43	296	100.407	5.339	55.756

II. SAFETY/RELIEF VALVE CHALLENGES

The attached report includes all actuations during the 1991 year. Two safety relief valves were cycled on January 13, 1991 per the surveillance test program in attempt to reset them. These valves had just been installed in the plant during refuel outage #2 which ended January 6, 1991. The cycling of these valves to reset them was unsuccessful. The tailpipe temperatures remained at approximately 225°F. The next time any SRV's were cycled was on December 22, 1991, following a manual reactor scram. The cycling of the SRV's was to decrease the pressure in the reactor. The attached sheet lists the valves actuated and the times. There were a total of 50 actuations. All valves were cycled by the manual mode in the Control Room. This information is submitted in accordance with Technical Specification 6.9.1.5.b of Appendix A to Operating License NPF-58 for the Perry Nuclear Power Plant, Unit 1.

Included as a SRV "challenge" are the following actions/events.

1. The SRV being physically operated by the manual control switch in the control room.
2. The SRV automatically lifts per design pressure limit when installed in the plant.

SAFETY RELIEF VALVE CHALLENGES DURING 1991

VALVE NUMBER	TIME	VALVE NUMBER	TIME
DATE-01/13/91		DATE-01/13/91	
1B21-F051D	0330	1B21-F047G	0518
DATE-12/22/91		DATE-12/22/91	
1B21-F041A	0315 0345	1B21-F041B	0311 0338 0508
1B21-F041C	0302 0332 0453	1B21-F041D	0241 0326 0358 0415
1B21-F041E	0340 0520	1B21-F041G	0233 0323 0353
1B21-F041F	0313 0341 0530	1B21-F047B	0225 0321 0351
1B21-F041K	0254 0330 0430	1B21-F047C	0251
1B21-F047D	0305 0334 0454	1B21-F047G	0258 0331 0353 0433
1B21-F047F	0228 0322 0353	1B21-F051A	0246
1B21-F051B	0237 0325 0355	1B21-F051C	0318 0347 0630
1B21-F051D	0221 0319	1B21-F051G	0309 0335 0337 0455

III. REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSIS

During 1991 there were no occurrences in which the Technical Specification 3.4.5, "Specific Activity," Limiting Condition for Operation was exceeded. This information is submitted in accordance with Technical Specification 6.9.1.5.c.

IV. SILICONE SEALANT INSPECTIONS

During 1991, no refueling outages were performed, therefore no silicone sealant inspections were performed. This information is provided in accordance with the response to License Commitment 17 of USAR Appendix 1B (reference letter PY-CEI/NRR-0703 L, dated August 27, 1987 from Murray R. Edelman).