

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Turkey Point Plant - Unit 4

DOCKET NUMBER (2)

0 5 0 0 0 2 5 1 1 OF 0 2

PAGE (3)

TITLE (4)

Engineered Safety Feature Actuation - Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									N/A	0 5 0 0 0
0 6	0 4	8 4	8 4	0 1 0	0 0 0	7 0	5 8	4	N/A	0 5 0 0 0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)								
N		20.402(b)		20.406(c)		X		50.73(a)(2)(iv)		73.71(b)
POWER LEVEL (10)		1 0 0		20.406(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(iv)		73.71(c)
		20.406(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		50.73(a)(2)(viii)(A)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.406(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(B)		50.73(a)(2)(ix)		
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(ix)		50.73(a)(2)(ix)		
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)		50.73(a)(2)(ix)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

Jesus Arias, Jr., Regulation and Compliance Lead Engineer

TELEPHONE NUMBER

AREA CODE

3 0 5 2 4 5 - 2 9 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 4, 1984, with Unit 4 at 100% power, the 4A steam generator feedwater pump tripped due to low suction pressure. This occurred while closing the low pressure heater bypass valve. This caused a 30% turbine runback on the unit. Subsequently, the reactor tripped. The cause of the unit trip was not immediately apparent so a review of the events was initiated. The Digital Data Process System (DDPS) printout indicated that the trip was initiated by opening of the 'A' reactor trip breaker. This in turn tripped the turbine which de-energized reactor trip relays 9 and 10 and opened the 'B' reactor trip breaker. No apparent cause for opening of the 'A' reactor trip breaker could be identified. First out annunciators indicated that the trip was caused by steam flow greater than feed flow coincident with low steam generator level on 'C' steam generator, yet the DDPS printout had no indication of any steam generator reactor trip matrix being made up. Additional details are described in the text portion of this report. The health and safety of the public were not affected. Similar occurrences: None. Significant event notification made to NRCOC via ENS pursuant to 10 CFR 50.72(b)(2)(ii).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Turkey Point Plant - Unit 4	0 5 0 0 0 2 5 1 8 4 -	0 1	0 -	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 386A's) (17)

On June 4, 1984, with Unit 4 at 100% power, the 4A steam generator feedwater pump tripped due to low suction pressure. This occurred while closing the low pressure heater bypass valve. This caused a 30% turbine runback on the unit. Subsequently, the reactor tripped. The cause of the unit trip was not immediately apparent so a review of the events was initiated. The DDPS printout indicated that the trip was initiated by opening of the 'A' reactor trip breaker. This in turn tripped the turbine which de-energized reactor trip relays 9 and 10 and opened the 'B' reactor trip breaker. No apparent cause for opening of the 'A' reactor trip breaker could be identified. First out annunciators indicated that the trip was caused by steam flow greater than feed flow with low steam generator level on 'C' steam generator, yet the DDPS printout had no indication of any steam generator reactor trip matrix being made up.

Following the reactor trip, a RCS cooldown coupled with a decrease in pressurizer level and pressure were observed. Root causes were identified to be:

- 1) Momentary lifting of various secondary system safety relief valves.
- 2) Prolonged auxiliary feedwater flow to the steam generators.
- 3) Near absence of decay heat following the trip.

To rule out the possibility of a pressurizer safety having lifted during the transient, a containment entry was made on the same day. In the control room, temperature indicator (TI-4-467) was reading abnormally high with a temperature of approximately 350°F and pressurizer relief tank (PRT) level had a slight increase. The containment entry team took local contact pyrometer readings in the areas of pressurizer relief and safety tail pipes and found no abnormal readings which indicates that no safeties had lifted during the transient.

The acoustic valve flow monitor did not indicate any pressurizer safety valve flow. This monitor was checked for operability following the transient and was found to be satisfactory. It was subsequently discovered that TI-4-467 was malfunctioning and thus produced erroneous high reading on the 'B' pressurizer safety valve tail pipe temperature. The unit was stabilized and maintained at hot shutdown while investigations continued into the root cause for the reactor trip.

The 'A' reactor trip breaker, the steam generator reactor trip logics, and the DDPS computer inputs were checked with no abnormalities discovered. The 'A' reactor trip breaker was replaced and the unit returned to power operation at approximately 8:00 a.m., on June 5, 1984.



July 5, 1984
PNS-LI-84-227

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 84-10
Turkey Point Unit 4
Date of Event: June 4, 1984
Engineered Safety Feature Actuation - Reactor Trip

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. W. Williams, Jr.", followed by a large, stylized flourish or "L" shape.

J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
File 933.1 TP

IE22
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