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July 6, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

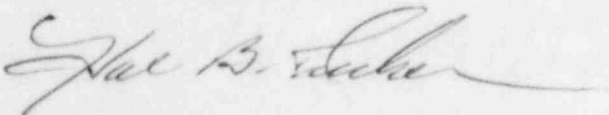
Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: Catawba Nuclear Station, Unit 1
Docket No. 50-413

Dear Mr. Denton:

As a result of discussions with NRC Staff reviewers, we are submitting a revision to Table 1.9-1 (page 16), TMI Concern II.K.3.17, Report on Outages of Emergency Core-Cooling Systems. This change is a restatement of a commitment which is discussed in the SER, Section 13.5.2. The attached page will appear in Revision 12 to the FSAR.

Very truly yours,



Hal B. Tucker

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Attachment

cc: Mr. James P. O'Reilly
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Response to TMI Concerns

II.K.3.10 PROPOSED ANTICIPATORY TRIP MODIFICATION

See Section 7.2.

II.K.3.11 JUSTIFICATION FOR USE OF CERTAIN PORV'S

See II.D.1

II.K.3.12 CONFIRM EXISTENCE OF ANTICIPATORY REACTOR TRIP UPON TURBINE TRIP

See Section 7.2

II.K.3.17 REPORT ON OUTAGES OF EMERGENCY CORE-COOLING SYSTEMS LICENSEE REPORT AND PROPOSED TECHNICAL SPECIFICATION CHANGES

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440.T.6 Duke will develop and implement a plan to compile ECC systems or components involved in outages. The plan will require a periodic report which contains (1) ECC system or components involved, (2) outage dates and duration of outages, (3) cause of the outage, and (4) corrective action taken. Test and maintenance outages will be included. The report will be reviewed and changes proposed to improve the availability of ECC equipment, if needed. This plan will be developed prior to fuel load.

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440.T.9

II.K.3.25 EFFECTS OF LOSS OF ALTERNATING-CURRENT POWER ON PUMP SEALS

At the Catawba Nuclear Station the reactor coolant pump seal water is supplied by the charging pumps and cooled by component cooling water. Nuclear service water in turn cools the component cooling water. In the event of a loss of off-site power, the component cooling water pumps, the nuclear service water pumps, and the charging pumps are all supplied with emergency power from the emergency diesel generators.

II.K.3.30 REVISED SMALL-BREAK LOSS-OF-COOLANT-ACCIDENT METHODS TO SHOW COMPLIANCE WITH 10 CFR PART 50, APPENDIX K

This item requires that the analysis methods used by NSSS vendors and/or fuel suppliers for small-break LOCA analysis for compliance with Appendix K to 10 CFR Part 50 be revised, documented, and submitted for NRC approval.

Westinghouse feels very strongly and Duke agrees that the small-break LOCA analysis model currently approved by the NRC for use on Catawba is conservative and in conformance with Appendix K to 10 CFR Part 50. However, (as documented in Letter OG-60, dated June 15, 1981, R. W. Jurgensen (Chairman, Westinghouse Owners Group) to P. S. Check (NRC), Westinghouse believes that improvement in the realism of small-break calculations is a worthwhile effort and has committed to revise its small-break LOCA analysis model to address. NRC concerns (e.g., NUREG-0611, NUREG-0623, etc.). This revised