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TUELECTRIC

February 27, 1992

William J. Cahill, Jr.
Group Vice President

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)
DOCKET NOS. 50-445 AND 50-446
SMALL BREAK LOCA MODE 4 OPERATION
SDAR CP-86-41 (INTERIM REPORT)

REF: 1) TU Electric Letter logged TXX-91296 from William J. Cahill, Jr. to NRC dated August 13, 1991

Gentlemen:

On May 28, 1986, TU Electric orally notified your Mr. I. Barnes of a deficiency involving evaluation of the effect of a small break Loss of Coolant Accident (LOCA) during Mode 4 operation. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

As of the last interim report (reference 1), the Westinghouse Owner's Group (WOG) had not issued a final report. Subsequently, Westinghouse topical report WCAP-12476, "Evaluation of LOCA During Mode 3 and Mode 4 Operation for Westinghouse NSSS," has been issued and was submitted to the NRC via WOG letter OG-91-61 on November 27, 1991.

The results of this study demonstrate that the current Emergency Core Cooling System (ECCS) and ECCS actuation system design for Westinghouse NSSS are adequate for Mode 3 and Mode 4 operation. In addition, analyses have shown that there is adequate operator action time to mitigate the consequences of a small break LOCA.

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TU Electric is reviewing the topical report to determine the plant specific actions for CPSES. An update delineating these actions will be issued by August 28, 1992.

Sincerely,

William J. Cahill, Jr.

By:



D. R. Woodlan
Docket Licensing Manager

DNB/dnb

c - Mr. R. D. Martin, Region IV
Resident Inspectors, CPSES (2)
Mr. T. A. Bergman, NRR
Mr. M. B. Fields, NRR