

AFFIDAVIT

My name is Steven Lockert. I am making this statement freely, without any threats or inducements, to Tom Devine, who is an attorney working with the Government Accountability Project.

I am making this statement because the evidence that was available to me indicates that the Pacific Gas and Electric Company was not responsible to the requirements of the Code of Federal Regulations regarding the reporting of defects and non-compliances. Further, that the M.W. Kellogg Company made false statements regarding a weld that failed in 1977 on the steam generator 1-2 nozzle indicating that the crack in the nozzle weld was not a generic problem; when in fact, there was documented evidence to the contrary on the steam generator 1-1 nozzle.

I contend that there was problems with the Kellogg Welding Procedure Specification # 200 (deviation from ANSI B31.1 pre-heat requirements for P3 materials of 175°F before welding,) problems with the 1-1 nozzle weld ( Field Weld 197 welded without preheat prior to 12/3/74 and again on 12/24/74,) and that there were problems with the Quality Assurance Program at the time because the nonconformance report that should of been generated apparently did not get reported per 10 CFR 21.21. Also, I contend that in 1977 after a crack in nozzle 1-2 was discovered re-examination of nozzle 1-1 revealed a similar crack that apparently did not get reported per 10 CFR 21.21 either.

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Response to PG&E Report 411-77.55

Subject: Failure Analysis of Cracked Field Weld No. 212, Diablo Canyon Unit 1, Steam Generator 1-2, Nozzle-To-Pipe Weld.

After review of the failure analysis report on F. W. 212 performed by PG&E's Department of Engineering Research, I have concluded that the report was put together with gross negligence or that a coverup activity is in progress to hide generic problems associated with the steam generator nozzle to feedwater pipe welded connections. F. W. 212 was performed by Pullman Power Products Corporation (M. W. Kellogg Company at that time) in May of 1974 and it is believed that the remaining welds on the other steam generator nozzles in Unit 1 was performed that same year. The history of the problems with these welds has extended from 1974 to 1977 to 1982 to today. PG&E's failure analysis of F. W. 212 has not put the matter to bed but has raised more questions about the acceptability of these weldments, the ethics of Pullman Power Products, and the ability of the licensee to meet the requirements of the Code of Federal Regulations for construction and operation of Nuclear Power Plants.

Problems with the failure analysis report are:



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- 1.) The Pullman welding procedure specification (WPS) supplied for the report has a revision date of 6/16/76. This is obviously not the WPS that was used for the actual weld performed in May of 1974.
- 2.) Pullman WPS 200 has an original date of 11/14/73, the date at which time the WPS was legally in effect. However, the accompanying Procedure Qualification Record (PQR) was performed on 12/28/73 over a month later. The WPS requires a PQR before the WPS can be written or used in the field.
- 3.) The nozzle material ASME SA-508 Class 2 is a P3 classified material under ASME Boiler & Pressure Vessel Code, Section IX requirements. Pullman's WPS 200 and accompanying PQR (P12b-P1-K1-F4-SMAW-6G) are for welds between P12b and P1 classified materials. The WPS for the nozzle welds would have had to be for P3 to P1 materials only.
- 4.) Pullman WPS 200 states that the preheat is only required for the SMAW portion of the weld. This procedure allows the tack and root welding to be performed without the benefit of a preheat. This procedure has an ANSI B31.1 Power Piping Code non-compliance written into it because the very definition of a preheat means "heating the base metal before

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a welding or cutting operation"(see paragraph 100.2, definitions, of ANSI B31.1 1977 edition.)

5.) Page 2 of WPS 200 has significant data on weld joint preparation written in so small that the data is illegible.

6.) Copies of the original Certificate of Analysis for the three electrode lots of E 8018-C3 show that one of the original lots L416F3AC was not even shipped until 2/20/75 nine months after the weld was performed.

7.) Figure 5 showing the crack in a macroexamination has the pipe and nozzle identifications reversed.

8.) The Certificate of Analysis for the E 70S-2 filler metal and E 70S-2 insert have not been provided.

9.) The Laboratory Sample 577.329 has been mistakenly labeled P12B material. Note that there is no P12B material listed under QW 422 of the ASME Code, Sec. LX (see page 27 of the PG&E report.)

10.) Table 1 of PG&E's report shows a preheat being performed on May 18, 1974 when the Pullman Swindell report specifically states that the preheat was only from May 22, 23, and 24 of 1974 (Table 1 is on page 9 of report.)

I believe that this is just more than sloppy report writing and a full understanding of how bad the welds really are can be

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attempted only after the data appropriate for PG&E's report is found for the period of time that the weld was made. It is true that F. W. 212 no longer exists because it has been replaced but that leaves the seven other nozzle to pipe welds that were performed with the same WPS that the original F. W. 212 was welded to.

Steam Generator 1-1 nozzle to pipe weld also has an interesting history. F.W. 197 was first performed prior to a Dec 3rd meeting between Mr. J. W. Ryan and Mr. P. J. Carosella, the then Pullman Construction Manager and Senior Safety Engineer for the Department of Industrial Relations of the State of CA, respectfully. Mr. Carosella makes mention of the fact that F. W. 197 had experienced a crack extending the circumference of the pipe because Pullman production had welded with out the use of preheat. The process sheet for the second try at F.W. 197, which by the way is not marked R1, is shown with the process sheet for F. W. 212 provided on page 33 of PG&E's report. Note that the preheat for the second try is not signed of by the MWK inspector and there is no reference to a preheat chart. Also note the inconsistencies in the inspection coverage between the two welds; the ANI checked for visual inspection but not the root pass on F.W. 197 but did just the opposite for F.W. 212. I think that the DR that covers why F.W. 197 was welded without preheat

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before Dec. 3 should be examined to make sure that the corrective action called for by Mr. Carosella in his Dec. 18th letter addressed to J. P. Runyan, W. M. Kellogg's QA/QC Manager was adequately established. Also, some explanation for the lack of preheat data available for the second attempt at F. W. 197 during Dec. 23 to Dec. 30 of 1974 must be provided. It occurs to me that both of these mistakes appear to be reportable per 10 C F R 50, paragraph 50.55e.

F. W. 197 was subsequently radiographed and the film read by an individual named Ken Beck on 1/28/75. Mr Beck noted that tungsten inclusions were distributed through about 75% of the weld. Mr. Beck did not note a drop thru that also had linearly oriented voids. Apparently, someone requested another radiograph because the weld was reradiographed but with wider film to include a repair made to the nozzle. Again the weld was accepted but this time with recognition of the burn thru on 2/11/75 by Mr. Shore.

The time frame for documentation of events now shifts to March 17, 1977 when the leak was discovered in F.W. 212. These events are documented in the M.W. Kellogg QA Report by J. P. Runyan dated 4/12/77. The radiograph for F.W. 197 was again reviewed at this time and a decision was reached to now remove the drop thru present on the inside of the pipe. The repair was made per DR 3370 and consisted of cutting a hole in the pipe

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and grinding out the burn thru. I think that it should be noted that the R. P. Runyan noted DR 3370 in his report dated 4/12/77 but that the letter addressed to Mr. R. H. Engelken of the USNRC Office of Inspection and Enforcement, Region V, written by a Mr. Philip A. Crane, Jr. makes no mention that a condition requiring repair had been found, in fact, Mr. Crane reported that PG&E's examinations revealed no rejectable indications for any of the four main steam and three feedwater welds as of 4/15/77.

In DR 3370 Mr. R. P. Runyan had noted that when he had reviewed Mr. Ken Beck's record for radiographic interpretation it was found that Mr. Beck had hired in on 1/2/74 but that he had not certified to the M. W. Kellogg radiographic Level II position until 8/9/74 and that he had accepted radiographs before he was certified to do so. However, instead of reporting the non-conformance via the QA system and notifying the licensee Mr. Runyan attributed the QA breakdown to an administrative error. A problem with Mr. Beck's work was apparent as early as 4/5/77 as evidenced by the interoffice correspondence from S. L. Engler to R. P. Runyan where Mr. Engler states that some of the welds must be repaired.

Although PG&E could find no rejectable indications in the steam generator nozzle to pipe welds as of 4/15/77 all four of the nozzle to pipe welds were reworked from the inside of the pipe during the period from 8/31/77 to 11/28/77. Cracks were found



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on the inside surface of F.W. 197, a grind and polish method was used to chase the cracks. One of the cracks was 5 3/8" long and required grinding below the minimum wall thickness before a liquid penetrant test would yield positive results. Of course after the minimum wall thickness had been violated a weld repair was in order. The grinding out of the cracks and the weld repair to F. W. 197 was apparently done to an interoffice correspondence initiated by a Mr. Don Geske instead of the direction of PG&E as indicated in step nine of DR 3484. The welding, of course, would have required a process sheet, filler metal requisitions, and the full awareness of a QC welding inspector. The DR 3484 fails to include these requirements in the recommended disposition and Mr. Don Geske was not qualified to do visual welding inspections.

Three cracks were found that required violation of minimum wall thickness before they could be successfully removed. Removal of the first crack left a groove 10" long that required grinding through about half of the pipes thickness before the crack on the nozzle side of the root pass was removed. I am not suprized that no one recognized that this crack was similar to the one that occured on F.W. 212 because that would indicate a generic problem. In fact, the people involved were very carefull not to refer to the defect as a crack, it was a

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linear indication right up to and even after the weld repair. The game is over because linear indications do not require weld repairs but cracks do.

A defect had been found, a crack extending 10" long and approximately half way through the thickness of the pipe. The crack originated on the nozzle side of the root pass under a roll over where the reentrant angle was probably less than 90 . Other cracks had been observed in the land surfaces of the nozzle and pipe counterbores. Lets compare the above to a quote from PG&E's report 411-77.55 "It is believed that small cracks initiated on the I. D. of the nozzle, weld, and pipe during the thermal cycling that occurred during preheating. These small cracks originated at convenient stress risers such as grinding scratches and regions of lack of fusion and weld bead rollover." Mr. Runyan could not see the similarities between the two because he had already made up his mind about the failure of F. W. 212 back in April. Mr. Runyan said in his summary to his QA Report on F.W. 2.4 "It is my believe that the crack was peculiar to F.W. 212 only and not of a generic nature. Therefore, at this time we are assuming that no further repair will be required and that when the disposition of D. R. 3370 is completed the subject will be closed."

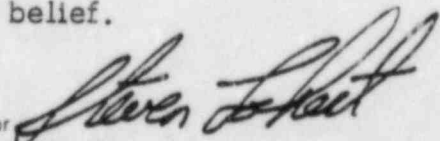
Defects had been found in steam generator nozzle to pipe welds that had been fully inspected and accepted. F.W. 197 and

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F.W. 244 revealed cracks on the nozzle side of the root similar to the crack that initiated the failure of F.W. 212. These cracks are defects which, if they were left uncorrected could have adversely affected the safety of the plant. The defects should have been reported to the Commission. I believe that a break down in Quality Assurance of construction has occurred because the welds had been accepted and had been put in service without discovering the defects. Additionally, after the failure of F.W. 212 when a 100% reinspection of radiographs on Class 1 welds had revealed problems in previous interpretations, when F.W. 197 required repair in April, and when extensive repairs had been made to all the nozzle to pipe welds in Unit 1 the NRC had not been notified as required by 10 CFR 21 and/or 10 CFR 50.55e. I believe that PG&E's failure analysis of F.W. 212 is shoddy work, I believe that there has been an attempt to fix the mistakes on the sly and that there has been purposeful withholding of information from the commission.

I have read the above 10 page statement and it is correct and true to the best of my knowledge and belief.

STATE OF CALIFORNIA  
COUNTY OF SAN LUIS OBISPO  
on 4/10/84 before me, the undersigned, a Notary Public in and for  
said State, personally appeared STEVEN LOCKERT

  
Steven Lockert

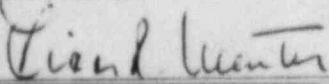
personally known to me (or proved to me on the basis of satisfactory evidence) to be the person(s) whose name(s) is/are subscribed to the within instrument and acknowledged to me that s/he/they executed the same.

WITNESS my hand and official seal.

(10 of 10)



Signature



(This area for official notarial seal)

LIST OF REFERENCED MATERIAL

ITEM	DATE	SUBJECT
1.	12/18/74	Crack in F.W. 197.
2.	3/23/77	D.R. 3370 (Review of Radiographs Resulting from Crack in F.W. 212.
3.	4/5/77	Letter from S. Engler to P. Runyan regarding K. Beck.
4.	4/12/77	M.W. Kellogg QA Report on Crack in F.W. 212.
5.	4/15/77	Letter from P. Crane to NRC, Office of IE.
6.	8/26/77	PG&E Failure Analysis of F.W. 212.
7.	6/23/77	Pullman Swindell Metal-lurgical Exam of F.W. 212
8.	8/26/77	D. R. 3453 Internal In-spection and Weld Repair of F.W. 244.
9.	10/7/77	D.R. 3484 Internal In-spection and Weld Repair of F.W. 197.
10.	11/8/82	D.R. 4662 Failure to have a PQR for MT Examination of F.W. 197 and F.W. 244
11.	10/6/75	Personnel Testing Record of Mr. D. R. Geske showing no examination scores for Magnetic Particle Testing

DEPARTMENT OF INDUSTRIAL RELATIONS

SIGN OF INDUSTRIAL SAFETY  
GOLDEN GATE AVENUE, SAN FRANCISCO

December 18, 1974



Address reply to:

EXHIBIT 1 TO  
ATTACHMENT 1

The W. W. Kellogg Company  
P. O. Box 367  
Avila Beach, CA 93242

Attention J. P. Runyan, QA/QC Manager

Subject: Quality Assurance at Diablo Canyon  
Nuclear Plant Construction site

Gentlemen

On December 3rd I discussed with you and Mr. J. W. Ryan the recent removal by your Production Personnel of the thermowells from the Reactor Coolant Loop, which had to be removed because they had been improperly installed by your Company. A Hold Tag had been applied at each location and Deficiency Report 2437 had been issued which outlined the corrective work and also clearly stated that the Quality Assurance Manager was to be notified before beginning any corrective work. Production failed to notify you and failed to follow the D. R. In doing so they violated the Non-Conformance Section of your Quality Assurance Manual. KFP-10 of your QA Manual assigns to you the responsibility to assure that its provisions are adhered to.

You will recall that at the same time the repair of Steam Generator 1-1 feedwater nozzle was taking place. A crack extending circumferentially 360° had been caused by your welders when they welded without the use of preheat. The welders had been assigned to the job by their supervision before you were in possession of a qualified welding procedure for P 1 to P 12-B Materials.

The installer in possession of the proper ASME symbol stamp and Certificate of Authorization is required to operate under a controlled system by NA 4451 of Section III of the Code. We would therefore request that measures be taken to assure that conditions that caused the violations will be promptly identified and corrected to preclude repetitions. It is also requested that, in accordance with NA 4800, the identification and cause of failures affecting quality and the corrective action be documented.

Very truly yours

P. J. Carosella  
Senior Safety Engineer

/as

cc J. W. Ryan, Construction Manager  
J. S. Gaylor



## THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

D.R. NO. 3370 Rev II  
ISO. NO. 500146 Rev I  
UNIT NO. 1  
CODE NO. N/A

## DISCREPANCY REPORT

CUSTOMER: Pacific Gas & Electric SPEC. NO. 8711 DATE: 3/23/77  
PROJECT: Diablo Canyon JOB NO. 7177 INSPECTOR: J. P. Runyan

DISCREPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle to pipe

## EXPLANATION OF DISCREPANCY:

As the result of a crack in F.W. #212 at the Feedwater nozzle on Steam Generator 1-2 (Ref. D.R. 3366), a review of the radiographs of the same welds on the remaining three (3) generators was performed by the N.R.C. In addition, the nozzle to pipe welds on the main steam leads were reviewed.

The review revealed a questionable indication in F.W. #197 on the feedwater lead to Steam Generator 1-1. The indication appears to be a burn through which contains linearly oriented voids. The accompanying radiographic report accepted the weld but did not indicate the presence of the burn through. Further investigation revealed that a second set of radiographs, shot at a later date, exist. This set also shows the burn through which was recorded on the radiographic report as acceptable (see copies attached).

(Continued on Page 2)

## RECOMMENDED DISPOSITION:

See Page 2Approved By: M.W.K. Field Q.A. Mgr. J. P. Runyan Date 3/23/77 Customer R. D. Ely Date 3/23/77 REF. PGE DR 286FINAL DISPOSITION: ☐ In Accordance With Above☐ Other (explanation and approval required): 6/22/77 JPK

Work Completed Insp: \_\_\_\_\_ Date: \_\_\_\_\_

Work Completed Insp: JPK Date: 6/22/77

## EXPLANATION (IF NECESSARY):

Revision I See page 3

Revision II See page 3

M.W.K. Field Q.A. Manager J. P. Runyan Date 6/20/77 Customer R. D. Ely Date 6/20/77 DR 286M.W.K. Field Q.A. Manager J. P. Runyan Date 3-30-77 Customer R. D. Ely Date 3/31/77 DR 286STEPS TO PREVENT RECURRENCE ☐ Not ApplicableTo be provided after further review.See Page 3Field Q.A. Manager JPKDISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other \_\_\_\_\_  
☒ Customer ☐ Receiving ☐ Field Inspector ( \_\_\_\_\_ )

ATTACH SKETCH IF NECESSARY

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

## DISCREPANCY REPORT

D.R. NO. 3370 *Rev II*  
 ISO. NO. 500146  
 UNIT NO. 1  
 CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 3/23/77  
 PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: J. P. Runyan *JPR*

DISCREPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle to pipe

### Explanation Of Discrepancy:

To determine the severity of the indications in the burn through, we have made additional radiographs of the burn through area. These radiographs indicate that the voids are probably contained within the burn through and do not extend into the weld. There is no indication of propagation of the indications.

U.T. of the weld by P.G.&E. does not indicate any indication within the weld which is outside the acceptance criteria.

Attachments 1, 2 & 3

### Recommended Disposition:

To eliminate the possibility of a potential problem at a later date, it is recommended that the burn through be removed.

The following procedure is to be followed for removal and reinspection:

1. Weld a 2" threaded coupling to the pipe as shown on the attached sketch using weld procedure 88/89 or 92/93. (FW # 827)
2. M.T. per ESD-209.
3. Post weld heat treat the weld - ESD-218.
4. Perform a hydrotest on the weld prior to drilling into the pipe.
5. Following satisfactory completion of the hydrotest, cut the hole through the pipe wall with a hole saw (Ref. ESD-230) NOTE: Additional reinforcement of 2" opening is not required per P.G. & E. calculations
6. Using an extended shaft grinder with a cone shaped rock, grind the burn through flush with the pipe surface.
7. Radiograph the area to assure complete removal of unacceptable indications.
8. Install threaded plug and seal weld (F.W. #828).

This installation is to be made in accordance with the National Board Inspection Code and ASME Section I

Reviewed by A.N.I. R.L. Sanderson *3/24/77*

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ATTACH SKETCH IF NECESSARY

## THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

DISCREPANCY REPORT

Rev II

D.R. NO. 3370 Rev I  
 ISO. NO. 500146  
 UNIT NO. ONE  
 CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 3/23/77  
 PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: J.P. Runyan

DISCREPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle at pipeREVISION IThe following action was taken.

Add to "Recommended Disposition" the following steps.

A. To determine if there exists an ambiguity with other welds accepted by the M.W. Kellogg film interpreter who accepted F.W. 197, the following action was taken:

1. Review qualification records of this individual to assure compliance with SNT-TC-1A.
2. Review all radiographs which were accepted by the individual.

NOTE: The findings of the review will be reported as a separate report and attached to this D.R. before the D.R. is closed. No revision will be required to the D.R.

SUMMARYSTEPS TO PREVENT RECURRENCE

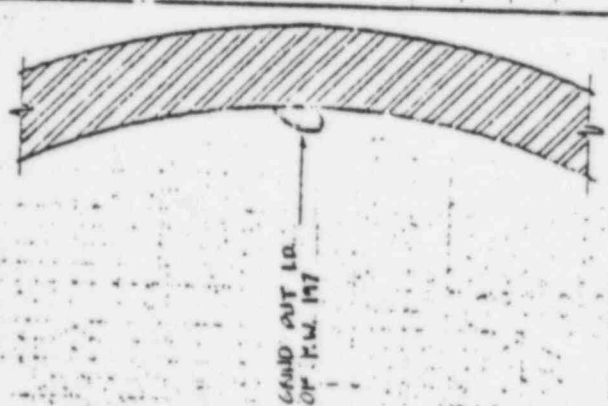
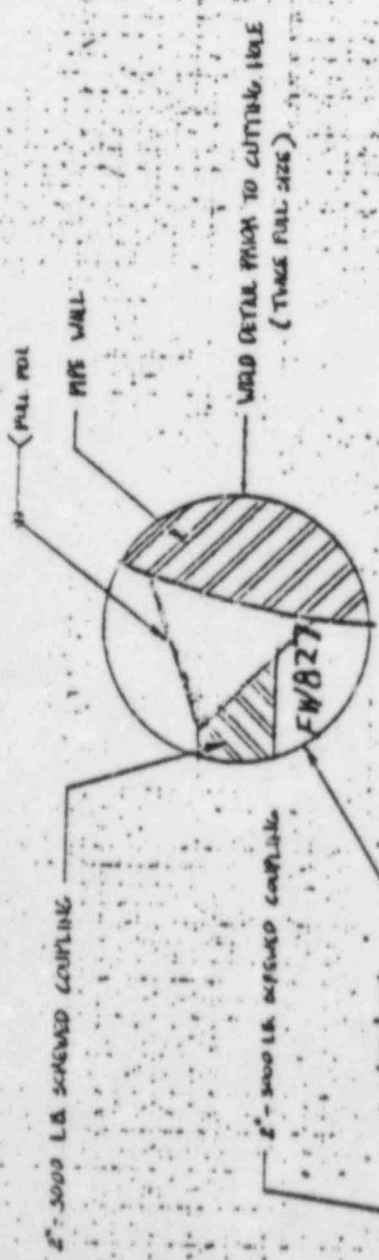
In addition to the above action it has been determined that a random review of accepted film from all other M. W. Kellogg interpreters was required. Twenty sets of film were reviewed from each of seven other interpreters. No rejectable indications were noted (See Q.A. report of film and documentation review dated 4-22-77 Attached).

Subsequent to the above a review of additional Unit I film was performed while investigating the accept/reject rate of welders. Another rejectable indication was noted in F.W. 346 Iso 1-4-500139 (Ref. D.R.3396). At this time P.G. & E. requested that a 100% film review be performed of all design class I radiographs in Unit I and II. This will be performed and documented in accordance with D.R. 3400 Rev. I for Unit I and D.R. 3409 for Unit II.

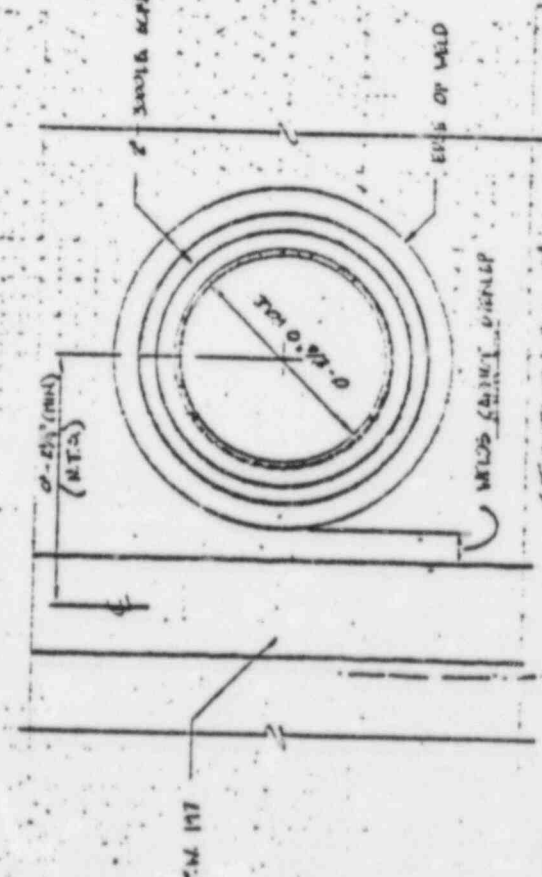
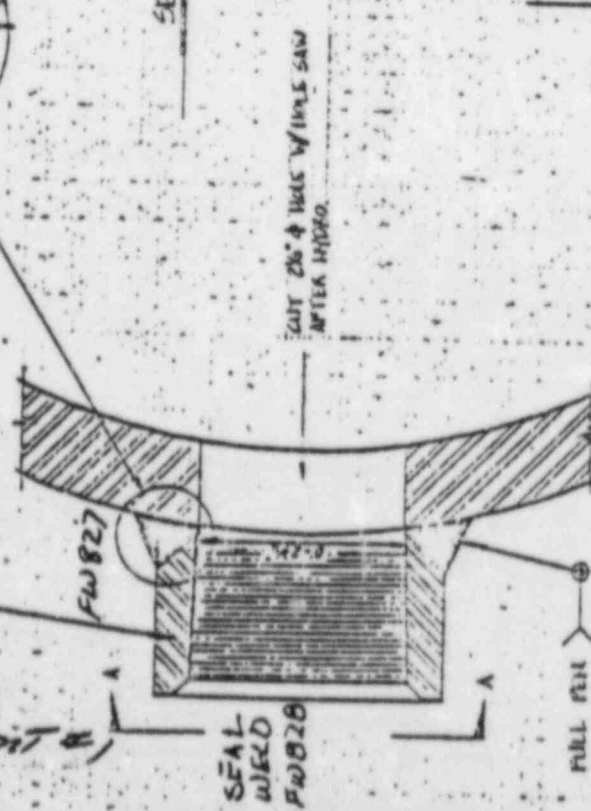
To further assure that all welds radiographed subsequent to this date are properly interpreted; each set of film will be interpreted and then reviewed by a second qualified film reader. Reference Q.A. instruction 101 Rev. I.

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☒ Customer ☐ Receiving ☐ Field Inspector ( \_\_\_\_\_ )

ATTACH SKETCH IF NECESSARY



SECTION THRU PIPE NEAR F.W. 197



SECTION "A-A"

DR 3370  
ATTACHMENT

PROCEDURE:

1. WELD ON 2\"/>



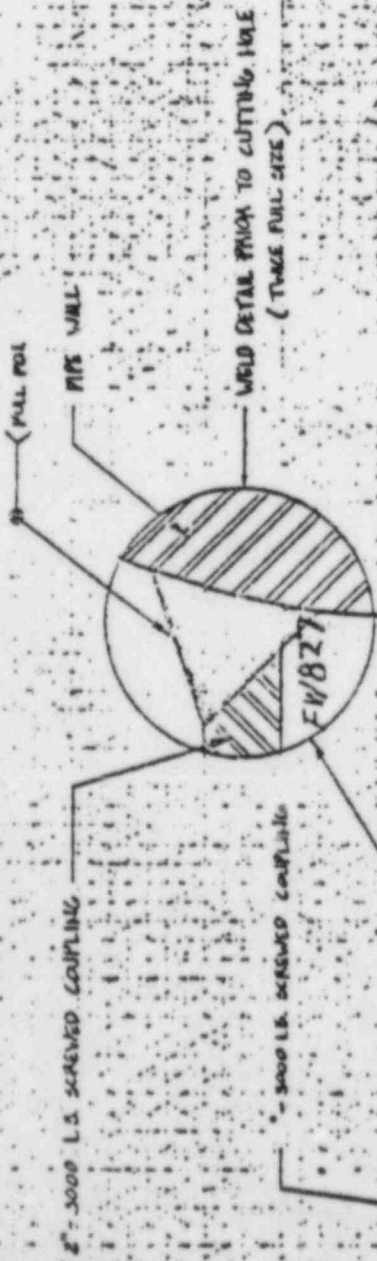




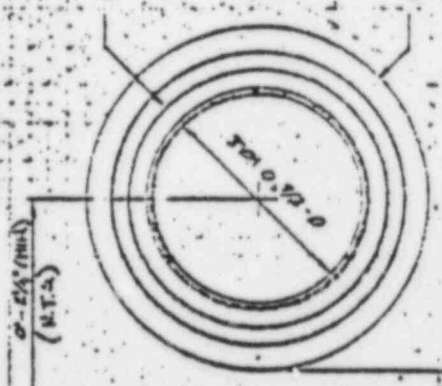
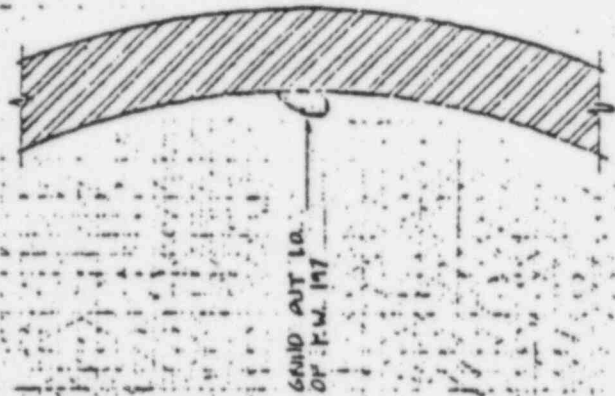


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NO.	REV.	DATE	BY	CHKD.	APP.	DESCRIPTION
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SECTION THRU PIPE NEAR F.W. 197



SECTION "A-A"

- PROCEDURE:
1. FIELD ON 2" - 3000 LB. SCREWED COUPLING
  2. WELD COUPLING
  3. CUT THRU PIPE WALL W/ 2 1/2" WIDE SAW
  4. GRIND OUT 10. OF F.W. 197

DR 3370

ATTACHMENT #1

SEAL WELD W828

FULL PIPE







DIABLO CANYON NUCLEAR PROJECT  
JOB #7177  
SPEC #8711

QUALITY ASSURANCE REPORT  
OF  
ACCOMPLISHED JOB TASKS  
(Ref: DR 3370)

THE M. W. KELLOGG COMPANY  
AVILA BEACH, CA

PREPARED BY

Donald R. Geske

D. Geske  
NDE Supervisor

APPROVED BY

J. P. Runyan

J. P. Runyan  
Field QA/QC Manager



QUALITY ASSURANCE REPORT  
OF  
ACCOMPLISHED JOB TASKS  
(Ref: DR 3 70)

PURPOSE

To document all job tasks performed as result of deficiencies noted during review of radiographic film required by DR 3370, steps to prevent recurrence.

SCOPE

This report is restricted to those items listed in our letter to the customer dated April 11, 1977 (see Attachment #1), and supplements our previous report by J. Runyan dated 4-22-77, Subject: Quality Assurance Report of Documentation and Radiographic Film Review.

FILM REVIEWED

K. BECK

UNI /SYST/LSO

FW

1-4-68	803
1-4-150	1051
1-7-1	*201, 204R1
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1-8-71	547
1-8-72	550, 552, 554, 2352B
1-8-349	1312R1
1-8-705	1830
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 1-9-42  
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 1-9-209  
 1-9-216  
 1-9-23  
 1-04-500136  
 1-04-500137  
 1-03-500146  
 1-03-500146  
 2-03-75  
 2-04-2  
 2-04-4  
 2-08-8  
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 2-08-25  
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 2-10-10  
 2-10-12  
 2-14-63  
 2-04-501014

141, 145R1  
 \*155, 158R1  
 173, 267, 268  
 182, 183  
 280, 281  
 283R1  
 250  
 533, 536  
 104  
 157  
 102, 20R1  
 253R1  
 279R1  
 228  
 229  
 532  
 15, 22, \*28, 29  
 41, 54, 55  
 49  
 \*111  
 \*185C, 185D  
 222, 233  
 \*243  
 249, 249A, 2486, 2487,  
 \*390  
 266, 267, 270  
 272  
 1479  
 78, 385  
 110  
 \*532R2  
 351

R. BARBER

UNIT/SYST/ISO

1-7-1  
 1-7-10  
 1-7-2  
 1-7-17  
 1-7-18  
 1-7-15  
 1-14-81  
 1-9-38  
 1-9-5  
 1-9-20

FW

1, 2  
 5R2  
 7  
 78, 79  
 80  
 68  
 564, 568A, 568, 579  
 230, 231  
 48, 47, 45, 46R1  
 150, 151

M. SHORE

UNIT/SYST/ISO

1-7-5  
 1-7-8  
 1-7-21  
 1-7-16  
 1-9-18  
 1-9-19  
 1-7-29

FW

25, 24, 295, 26, 23  
 22  
 100, 102, 103, 104, 105, \*99  
 74, 75  
 124  
 138, 137, 142  
 \*188, 189

R. SEARSUNIT/SYST/ISOFW

2-3-14	165
2-4-12	138
2-4-501016	323.
2-8-61	445A
2-8-480	848
2-9-15	117
2-9-32	196, 197
2-12-8	91
2-12-11	119
2-14-62	520, 814, 843, 861

M. COSTLEYUNIT/SYST/ISOFW

1-7-1	209, 211, 215, 213, 4
1-7-8	39, 40, 242
1-7-24	126, 129, 124
1-7-28	180
1-8-1	13
1-10-5	56E, 56F
1-14-35	181
1-14-81	691, 574, 583A, 576

J. RUNYANUNIT/SYST/ISOFW

2-3-19	232, 237, 427
2-3-501023	333
2-4-27	294
2-4-501018	158
2-7-15	106, 115
2-8-9	55, 56, 57, 567, 569
2-8-16	105, 576, 577
2-8-63	451
2-8-65	650
2-14-62	512

E. MARTINDALEUNIT/SYST/ISOFW

1-3-500146	499, 500, 501, 502, 503, 504, 505, 506, 507, 508, 509
1-3-6	349, 348
1-7-14	62, 63
1-7-15	67
1-7-17	77, 76
1-7-19	82, 83
2-4-47	*501
2-8-60	437A
2-9-32	302
2-14-62	505, 517, 818

\*Review required job tasks.

FILM RESHOT FOR QUALITY/CLARITY

<u>UNIT/SYST/ISO</u>		<u>FW</u>	<u>COMMENT</u>
1-04-500136	SG 1-1	255	Compared to original film - no defect noted.
1-04-500137	SG 1-2	282	Compared to original film - no defect noted.
1-04-500138		320	Compared to original film - no defect noted.
1-04-500139		348	Compared to original film - geometric difference noted, see DR 3423.
1-07-1		201	Drop thru shot in cross-section and determined to be within code after customer review 5-19-77.
1-07-21		99	Reshot for lighter density - no defect noted 5-27-77.
1-07-22		106	Reshot view 3-4. Original film missing - no defects noted 5-27-77.
1-07-25		170	Reshot due to rough surface - FW ground for inservice - no defect noted 5-18-77.
1-07-29		188	Reshot view 2-3-4 for full coverage - no defect noted 5-16-77.
1-09-3		388	Unconsumed insert noted during film review. Repair included with DR 3400.
1-09-16		279	Reshot view 0-1-2 to verify root fusion - no defect noted 5-25-77.
1-09-18		123	Reshot view 8-9-0 to verify root fusion - no defect noted 5-25-77.
1-09-21		155	Reshot view 4-5-6 to verify removal of surface condition - no defects noted 5-26-77.
2-04-501013		347	Compared to original film - no defects noted.
2-04-501014		364	Compared to original film - no defects noted.
2-04-501015		379	Compared to original film - no defects noted.
2-04-501016		329	Compared to original film - no defects noted.
2-04-47		501	Compared to original film - no defects noted.
2-08-17		111	Reshot view adjacent to repair area - no defects noted 5-13-77.
2-08-25		185C	Repair completed and accepted 5-13-77.
2-08-32		243	Repair completed and accepted - DR 3383, 5-20-77, hydro open.
2-08-33		390	Repair completed and accepted - DR 3385, 5-26-77, hydro open.
			Repair completed and accepted - DR 3384, 6-3-77, hydro open.

2-14-63

532

Repair completed and accepted -  
DR 3382, 6-3-77, hydro open.

OTHER JOB TASKS

UNIT/SYST/ISO

FW

COMMENT

1-03-500146

828

Penetrant tested seal weld at  
coupling installed to repair  
FW 197.

2-04-2

28

Visual inspection verified  
surface condition within code.

CONCLUSION

The above data reflects all actions completed as result of film review required per DR 3370 and constitutes final action as indicated. Defects discovered during film review generated the requirement for 100% radiographic film review. DR 3400 documents review of Unit I and DR 3409 concerns Unit II.

1 Attachment



*file*  
INTEROFFICE CORRESPONDENCE

TO J. HOLLEY

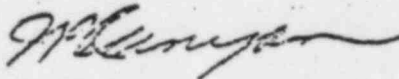
DATE APRIL 22, 1977

FROM J. P. RUNYAN

SUBJECT PROPOSED JOB TASKS - D.R. 3370

As result of Radiographic Film Review, conducted per D.R. 3370, the attached lists present our proposed action.

Request your review and comments at the earliest possible date.



J. P. Runyan  
Field Q.A./Q.C. Manager

JPR/mk

Attachments

## UNIT II

T - SYSTEM

ISO	P.W.	WORK REQUIRED	PREPARATION REQUIRED
2-04-501013	347	Reshoot Mainsteam	1. Scaffold at top of steam generator. 2. Not insulated.
	NOTE:	Recommend R.T. from inside of steam generator and P.T. of I.D. of welds.	
2-04-501014	364	Reshoot Mainsteam	1. Scaffold at top of steam generator. 2. Not insulated.
	NOTE:	Recommend R.T. from inside of steam generator and P.T. of I.D. of welds.	
2-04-501015	379	Reshoot Mainsteam	1. Scaffold at top of steam generator. 2. Pull insulation.
	NOTE:	Recommend R.T. from inside of steam generator and P.T. of I.D. of welds.	
2-04-501016	329	Reshoot Mainsteam	1. Scaffold at top of steam generator. 2. Pull insulation.
	NOTE:	Recommend R.T. from inside of steam generator and P.T. of I.D. of welds.	
2-04-2	28	Visual for surface Linear	1. Scaffold or ladder. 2. Pull insulation. 3. Match linear or; 4. Reshoot.
2-04-47	501	Reshoot adjacent view to repair area.	1. Modify existing scaffold.
2-08-17	111	U.T. for lack of Penetration.	1. None
2-08-25	185C	Repair gas holes	1. None
2-08-32	243	Repair gas holes	1. Erect scaffold on top of liquid hold-up tank 2-1.
2-08-33	390	Repair unconsumed ring	1. None.
2-14-63	532	Repair porosity	1. Scaffold platform for access.

NDE WORK D.R. 3370

## UNIT I

UNIT - SYSTEM  
ISO

F.W.

WORK REQUIRED

PREPARATION REQUIRED

1-04-500136

255

Reshoot Mainsteam

1. Erect scaffold on top of steam generator.
2. Pull temporary insulation.

NOTE:

Recommend R.T. from inside of Steam Generator and P.T. of I.D. of welds.

1-04-500137

282

Reshoot Mainsteam

1. Erect scaffold on top of steam generator.
2. Pull temporary insulation.

NOTE:

Recommend R.T. from inside of Steam Generator and P.T. of I.D. of welds.

1-04-500138

320

Reshoot Mainsteam

1. Erect scaffold on top of steam generator.
2. Pull temporary insulation.

NOTE:

Recommend R.T. from inside of Steam Generator and P.T. of I.D. of welds.

1-04-500139

348

Reshoot Mainsteam

1. Erect scaffold on top of steam generator.
2. Pull temporary insulation.

NOTE:

Recommend R.T. from inside of Steam Generator and P.T. of I.D. of welds.

1-03-500146

828

Penetrant Test  
A.I. Witness

1. Scaffold in place
2. Pull temporary insulation.
3. Power brush to clean.

1-07-1

201

R.T. Drop-Thru at  
view 3-4

1. No scaffold required.
2. Pull insulation.

1-07-21

99

Reshoot - Original  
Density 4.0+

1. Scaffold for access.
2. Pull insulation.

1-07-22

106

Reshoot - Original  
view 3-4 missing

1. Scaffold for access.
2. Pull insulation.

1-07-25

170

Reshoot - Original  
Rough surface

1. No scaffold required.
2. Pull insulation.

1-07-29

188

Reshoot - Original  
2-3-4 missed weld.

1. No scaffold required
2. Pull insulation.

1-9-3

338

Repair insert

1. No scaffold required.
2. Not insulated
3. R.T. repaired area.

NDE WORK D.R. 3370

## UNIT I

## UNIT - SYSTEM

ISO

F.W.

WORK REQUIRED

PREPARATION REQUIRED

1-09-16

279

U.T. for possible  
lack of fusion

1. No scaffold required.
2. Not insulated.

1-09-18

123

U.T. for possible  
lack of fusion

1. Scaffold in place.
2. Not insulated.

1-09-21

155

Reshoot - Original  
with possible slag

1. No scaffold required.
2. Pull insulation.



DIABLO CANYON NUCLEAR PROJECT

JOB# 7177

SPEC.# 8711

QUALITY ASSURANCE REPORT  
OF  
DOCUMENTATION AND RADIOGRAPHIC FILM REVIEW  
( REF. D.R. 3370)

THE M. W. KELLOGG COMPANY  
AVILA BEACH, CA.

PREPARED BY: J. P. Runyan  
J. P. RUNYAN  
FIELD Q.A./Q.C. MANAGER

DATE 4/22/77

QUALITY ASSURANCE REPORT  
OF  
DOCUMENTATION AND RADIOGRAPHIC FILM REVIEW  
(REF. D.R. 3370)

PURPOSE

To perform a comprehensive review of radiographic film and documentation related to field weld 212 and other similar welds.

SCOPE

To review the weld procedure, welder qualification records, heat treatment records, material certifications, nondestructive examination personnel qualification and N.D.E. results of the above mentioned welds.

DISCUSSION

Because of the crack in F.W. 212 it was determined that a complete review of the history of the weld, including inspection and tests records, was in order. In addition, other welds of the same configuration and material combination should be reviewed to determine, if possible, if there exists a generic type problem. The result of that review is as follows.

WELD HISTORY REVIEW

Field weld 212 was fit-up and tack welded on Friday May 18, 1974. On Monday May 21, 1974 the tack welds and insert were removed. A new insert was installed on May 22, 1974. It is not documented as to why the original insert was removed, however, discussions with field inspectors who were on site at the time indicate that surface rust may have occurred over the weekend and the insert was removed to reclean the weld prep and install a new insert prior to consuming the ring.

The root pass was made on May 22, and accepted visually May 23, 1974. Welding proceeded following Q.C. acceptance and was completed on May 24, 1974.

The weld surface was ground and final visual inspection completed on May 28, 1974.

Other than replacing the insert no unusual or out of the ordinary circumstances were recorded.

PREHEAT AND POSTHEAT

A review of the heat charts on F.W. 212 indicate that the weld area was preheated to 200° F. min. prior to tack welding and subsequent welding.

The weld was preheated on May 22, 23, and 24. The heat was turned off at the end of each shift allowing the weld to cool.

DR 3370

Ref: Mat L

5/27/77

UNIT I FEEDWATER		WELD No.	WELDER	FITTER	INSPECTOR
500146	S.G 1-1	FW 197	GX	926 117 - GRIND ONLY	BILL GUEST
500146	S.G 1-2	FW 212	LB	671 415-GRIND 117. PRE-HEAT	" "
500146	S.G 1-3	FW 229	SI	58 214-PREHEAT	" "
500146	S.G 1-4	FW 244	SI	1339 671 816	" "
		FW 244 R1	EF	434	A. CUBBAG BILL GUEST

UNIT I MAIN STEAM		WELD No.	WELDER	FITTER	INSPECTOR
500136	S.G 1-1	FW 255	SI	58 415	BILL GUEST
500137	S.G 1-2	FW 282	SI & GX	58 415	" "
500138	S.G 1-3	FW 320	SI & GX	58 929 1113	" "
500139	S.G 1-4	FW 348	SI & GX	58	" "
		FW 348 R-1	EF	58	" "

WELDER'S	FITTER'S
GX (411) D. KING - STILL HERE	926 - EVANS TERMINATED 3/19/76
LB (422) K. STALKER - TERMINATED 11/3/74	671 SCHMIDT " 11/8/74
SI (896) HERRING " 6/19/75	58 LOGUE " 6/3/75
EF (84) R. CHAPMAN " 7/9/76	214 HARDISON " 4/16/74
	816 WOLTMAN " 8/27/74
	1339 GERDES " 11/8/74
	434 BUCHANAN " 11/8/74
	117 TILTON STILL HERE
	415 GOTT STILL HERE
	929 ERNE 6/28/74
	1113 - ADDITIONAL 6/14/74

Following completion of the weld on May 24, the weld was allowed to cool and no further heating was performed until post weld heat treatment on June 24, 1974.

Preheat and post heat records from other welds of the same type indicate a similar history. NOTE: Other welds reviewed were the remaining three feedwater nozzle to pipe and all four main steam nozzle to pipe welds in Unit I and the main steam nozzle to pipe welds in Unit II. The feedwater nozzle to pipe welds are not welded in Unit II.

#### WELDING PROCEDURE AND WELDER PERFORMANCE QUALIFICATION RECORDS

Field Weld 212 was made using weld procedure number 200. The procedure was reviewed to assure compliance with ASME Section IX. No deviations were noted. The results of the procedure qualification tests were evaluated. Included were bend test's, tension test's, and Charpy V Notch test's. All results were acceptable.

Welder performance qualification records were reviewed. The records were in order and the welder was found to be properly qualified. In addition, records of other welds performed by the same welder were reviewed. It was determined that his performance record was good. There was no reason to suspect that welders capability or performance was below standard.

#### MATERIAL CERTIFICATIONS

Material Certifications of the weld material and pipe-side base material was reviewed for compliance with material specifications and job specifications, including supplementary requirements. All were found to be in compliance. The nozzle base material certifications were not available for review by M.W. Kellogg.

#### NONDESTRUCTIVE EXAMINATION PERSONNEL, PROCEDURES AND REPORTS

The original radiographs of F.W. 212 were reviewed. There was an area along the nozzle side of the root with greater density than other areas of the weld. This area is typical of other similar welds. The higher density is caused by the nozzle counterbore which is approximately  $\frac{1}{4}$ " wide. There was no evidence of linear indications in the area where the crack occurred.

Review of the radiographs of other similar welds did not show evidence of linear or crack-like indications. However, the radiographs of F.W. 197, which is the feedwater nozzle to pipe weld on steam generator number 1-1, indicated a drop thru which appeared to have linearly oriented voids contained within the drop thru. There were two sets of radiographs of F.W. 197. The reader sheet for one set listed the drop thru as acceptable. The other reader sheet accepted the weld but did not indicate that the drop thru was present.

To assure the soundness of the weld it was determined that the drop thru should be removed. This was accomplished in accordance with D.R. 3370. (Attached)



To further assure that the film interpreter, which had accepted the weld with the drop thru without entering it on the reader sheet, had not overlooked other similar or more serious indications, a complete re-view of all the film read by him was made. His qualification records were also reviewed. The results are as follows.

Mr. Ken Beck was employed by the M.W. Kellogg Company from January 2, 1974 to March 28, 1975. His resume and personnel records indicates he had sufficient past experience and training to qualify him as a Level II radiographer capable of film interpretation. He completed M. W. Kellogg R.T. exams and was certified to Level II on August 9, 1974.

A review of the films which were read by him indicated that he had reviewed and accepted film prior to August 9, 1974. It is assumed that this was an administrative error. To assure that the film were properly interpreted and accepted by a qualified reader, all of Beck's film was re-interpreted.

In general, the findings revealed that he frequently did not record all indications on the reader sheet. We found that he had reviewed one hundred twenty-two (122) sets of film, eight had indications which in our opinion should be repaired. These repairs will be performed as soon as the systems are made available. NOTE: Four of the repairs are in Unit I, Four are in Unit II. To further confirm our findings, the Q.A. Manager and Level III N.D.E. examiner from our Paramount plant was asked to review the film which we had reviewed. His review confirmed our interpretation, a copy of his report is attached.

In light of the fact that Beck had accepted welds with rejectable indications, it was determined that other film which had been accepted by other M.W. Kellogg interpreters should be reviewed.

Seven people, other than Beck, had reviewed film. Twenty sets of radiographs interpreted by each of the seven were selected randomly and reviewed. Of the one hundred forty (140) film reviewed no rejectable indications were noted. It was noted that occasionally acceptable minor indications were not recorded on the reader sheet. However all significant acceptable indications were apparently evaluated and recorded.

Other N.D.E. reports, (i.e. M.T. and P.T.) were reviewed and found acceptable. This included a review of personnel and procedure qualification records.

#### CONCLUSION

It is the writers opinion that the review as recorded above has covered all potential areas for problems in welding, heat treatment and N.D.E. related to the crack in F.W. 212.

It revealed a discrepancy in the interpretation of radiographs which has been corrected. Procedures are in effect to prevent recurrence of similar problems.

No further action is anticipated at this time.

# DISCREPANCY REPORT

COPY TO MILLER 2211030

TO: R. L. Lorenz From: D. Day

Page 1 of 1

Comp. Pullman Kellogg Cont. / Camp. I.D. No. U.R. 3370

Field No. 225

## INSPECTION SECTION

Discrepant Item: Belt 1, System 3, Steam Generator 1-1 Feedwater Control Field No. 107.

Explanation of Discrepancy: Radiography series revealed "drop through" with questionable indications. This was found after the radiograph and weld had been final accepted. For further details, see attached Pullman Kellogg U.R. No. 3370.

*J. B. Miller*  
Field Eng./Inspector

3-23-77  
Date

## CLASSIFICATION & DISPOSITION

Minor Variation ☐

Deviation ☒

Disposition: Reject. Repair in field in accordance with P-R U.R. 3370.

Following members of the Material Review Board concur with the above Disposition:  
(For Minor Variations, only Resident Engineer's signature required.)

*[Signature]*  
Name

3/24/77  
Date

*R. D. Miller*  
Resident Engineer

3-23-77  
Date

*[Signature]*  
Name

3/24/77  
Date

*Austin G. Miller*  
Responsible Engineer

3/24/77  
Date

*[Signature]*  
Supervising Engineer

3/24/77  
Date

## DISPOSITION ACCOMPLISHED

Remarks:

Field Engineer/Inspector

Date

## STEPS TO PREVENT REOCCURRENCE (Deviation Only)

Resident Engineer

Date

Copies: Attachments  
Resident Engineer, Chief Engineer

Report Completed

INTEROFFICE CORRESPONDENCE

EXHIBIT 3 TO  
ATTACHMENT 1

TO P. RUNYAN

DATE APRIL 5, 1977

FROM S. L. ENGLER

SUBJECT INTERPRETATION OF RADIOGRAPHS PREVIOUSLY INTERPRETED BY  
K. BECK (PULLMAN KELLOGG)

Reviewed 24 seams that were previously accepted by K. Beck. These seams were reviewed by other Pullman Kellogg personnel before I arrived. They broke them into two groups: Acceptable and Questionable.

During my review of the Questionable Radiographs I found a lack of understanding by the film interpreter of the governing acceptance standards, such as:

- A. Acceptable inclusions not noted on inspection report.
- B. Acceptable surface indications not marked on inspection report.
- C. Inclusions and surface conditions where accepted that should have been rejected.

In my opinion some of the seams will have to be repaired or at least re-radiographed. Some film can be accepted by comparison with the surface of the weld.

My review of the films interpreted by K. Beck that were re-reviewed by Pullman Kellogg personnel, and determined to be acceptable, consisted of a random sample. In the process of that review no rejectable film were found.

During the inspection I found that all other aspects of the program such as: Contrast, Sensitivity, Density and Processing were acceptable.

If you have any questions on the above report please contact me.

  
Stephen L. Engler

SLE/mk

cc: E. B. Curcio  
J. W. Ryan  
S. Handler

DIABLO CANYON NUCLEAR PROJECT  
JOB # 7177  
SPEC # 8711

QUALITY ASSURANCE REPORT

OF

Crack in Steam Generator Feedwater  
Nozzle to pipe weld  
(Generator No. 1-2)

THE M.W. KELLOGG CO.  
Avila Beach, Ca.

PREPARED BY J. P. Runyan  
J. P. RUNYAN  
Field Q.A./Q.C. Manager

DATE April 12, 1977



## QUALITY ASSURANCE REPORT

OF

### CRACK IN STEAM GENERATOR FEEDWATER NOZZLE TO PIPE WELD (GENERATOR NO. 1-2)

#### PURPOSE

To document events from the time of discovery of the leak until the completion of the repair.

#### SCOPE

This report covers the on site findings, the review of documentation including preheat and postheat charts, radiographs from the defective weld, the subsequent procedures for removing the defective area and performing the repair.

#### FINDINGS

On Thursday, March 17, 1977 a leak was observed in field weld 212, line K16-555, by P.G. & E. We were advised of the leak on that date.

The weld in question is where the 16" feedwater pipe ties into nozzle #4 on Steam Generator 1-2.

Visual observation revealed a weep type leak which occasionally sprayed a fine stream of water. When observed with a 10X magnifying glass there appeared to be a small intermittent linear indication approximately 3/8" in length in the center of the weld running around the pipe. Magnetic particle examination of the area with a D.C. converted yoke did not show evidence of a linear defect.

P.G. & E. requested that we grind the area. As grinding proceeded, the indication opened to reveal a linear defect approximately 2" long when the weld crown was flush with the pipe surface. At this point the grinding was stopped. We were requested to perform an Ultrasonic examination to determine the extent of the indication. The pipe temperature was approximately 180° F which made it impossible to perform U.T. with the standard transducers on site.

We were then instructed to "hold" until P.G. & E. Engineering Research arrived with their U.T. equipment and high temperature transducers. The weld was radiographed at this time. The radiograph revealed evidence of a linear indication which appeared to be approximately 6" in length.

On Friday, March 18, 1977 P.G. & E. Engineering Research arrived and performed Ultrasonic examination of the weld. They reported that there was a crack which appeared to extend approximately 2/3 of the distance around the weld. Based on these findings it was determined that the weld would be cut out and replaced.

## WELD REMOVAL

The weld was cut by grinding approximately 1/2" from the center line on the nozzle side and at F.W. 503 on the pipe side. The end of the pipe was then cut to remove a ring which included most of the weld and approximately 4" of pipe.

The piece was examined visually and by liquid penetrant on the O.D. and I.D. and a sketch made to reflect the observations. (Sketch Attached)

The piece was shipped to P.G. & E. research lab on March 20 for analysis.

## REPAIR

A piece of 16" pipe was removed from stores to replace the piece which was cut out. The pipe end preps, gamma hole and vent were machined in the P.G. & E. machine shop on March 19, 1977. The nozzle end prep at F.W. 212 and pipe end at F.W. 503 were ground in place on March 20. A liquid Penetrant examination was performed at that time to assure complete removal of any indications.

The new piece was moved into place and fit up on March 20. Preheat was applied prior to tack-up, (Ref. Chart # 547). After the fit up was approved by M.W. Kellogg Q.C. both roots were welded. Magnetic particle inspection was performed. Following magnetic particle acceptance two additional passes were welded in each weld.

Radiography was performed on March 21, 1977. The radiographs of F.W. 503 pipe to pipe were acceptable. F.W. 212, pipe to nozzle, had excessive porosity at the window closures. These areas were ground, rewelded and re-radiographed. One area had excessive porosity, the other had a linear indication approximately 1/2" long and an area which appeared to be suck back.

At this time, March 21, 1977, P.G. & E. Q.A. placed a "hold" on all work until their Engineering could review the total program and process appropriate paperwork. "This hold was not because of the difficulty in welding the window closures".

No work was performed on March 22, 1977.

On March 23, 1977 an on site meeting was held to review findings, work to date, and procedures for completion of the repair. Attending the meeting were representatives from P.G. & E. Engineering, Q.A., General Construction, Division, Westinghouse, M.W. Kellogg site Manager, Corporate Q.A. Manager, and the writer.

- A. It was determined that the procedure as outlined by D.R. 3366 was acceptable except that the preheat would be raised to 300° Min. for completion of the welding.
- B. The problem of making an acceptable closure weld was discussed. It was determined that the positive pressure maintained in the system was too high and that water remaining in the generator caused vapor to be carried out the window openings.

- C. P.G. & E. Division agreed to drain the generator and reduce the purge pressure until the closure welds could be made.

On March 24, 1977 the defective areas at the closures were ground out and Magnetic particle performed to assure complete removal. No welding was performed since P.G. & E. had not removed the Hold. Preheat was being maintained through the "hold" period.

On March 25, 1977 the hold was released. The windows were closed and radiograph made. Both areas were acceptable and welding continued. The weld was completed on the afternoon of the 25th. Radiographs were shot in the hot as welded condition. No rejectable indications were noted. Post weld heat treatment was then performed.

On March 26, 1977 following post weld heat treatment, the weld was ground, re-radiographed, liquid penetrant inspected and ultrasonically examined. No rejectable indications were noted. The vent line and gamma plug were completed, inspected and the system turned over to division for resumption of testing.

#### SUBSEQUENT INVESTIGATION

Because of the unknown origin of the crack it was determined that other welds of the same type and welded using the same procedure should be reviewed to determine if the defect could be a generic nature.

Each of the remaining three feedwater nozzle welds and the four Main Steam nozzle welds were ultrasonically examined. The original radiographs were reviewed and the feedwater nozzle welds were re-radiographed. There was no evidence of like indications in any of the welds and no evidence of change in any of the noted, acceptable, indications since the original inspections.

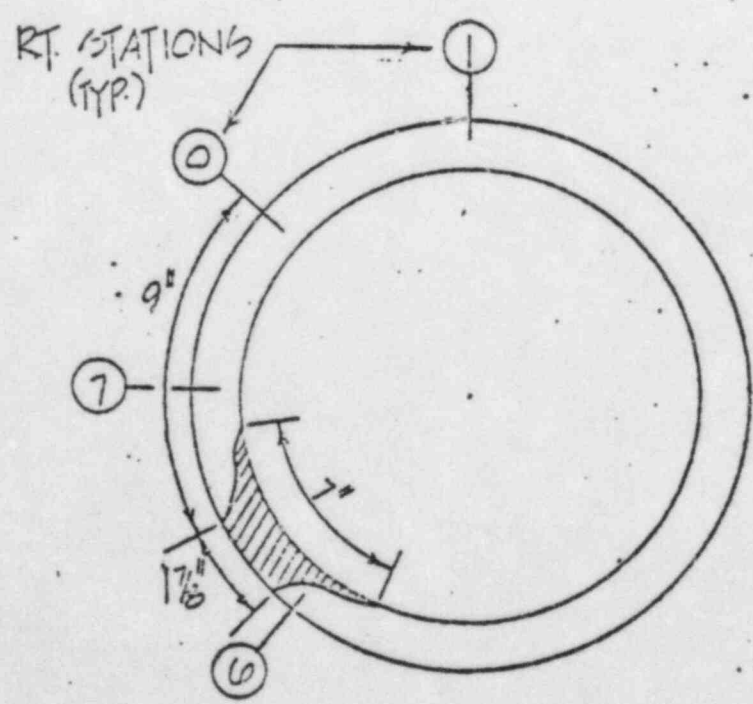
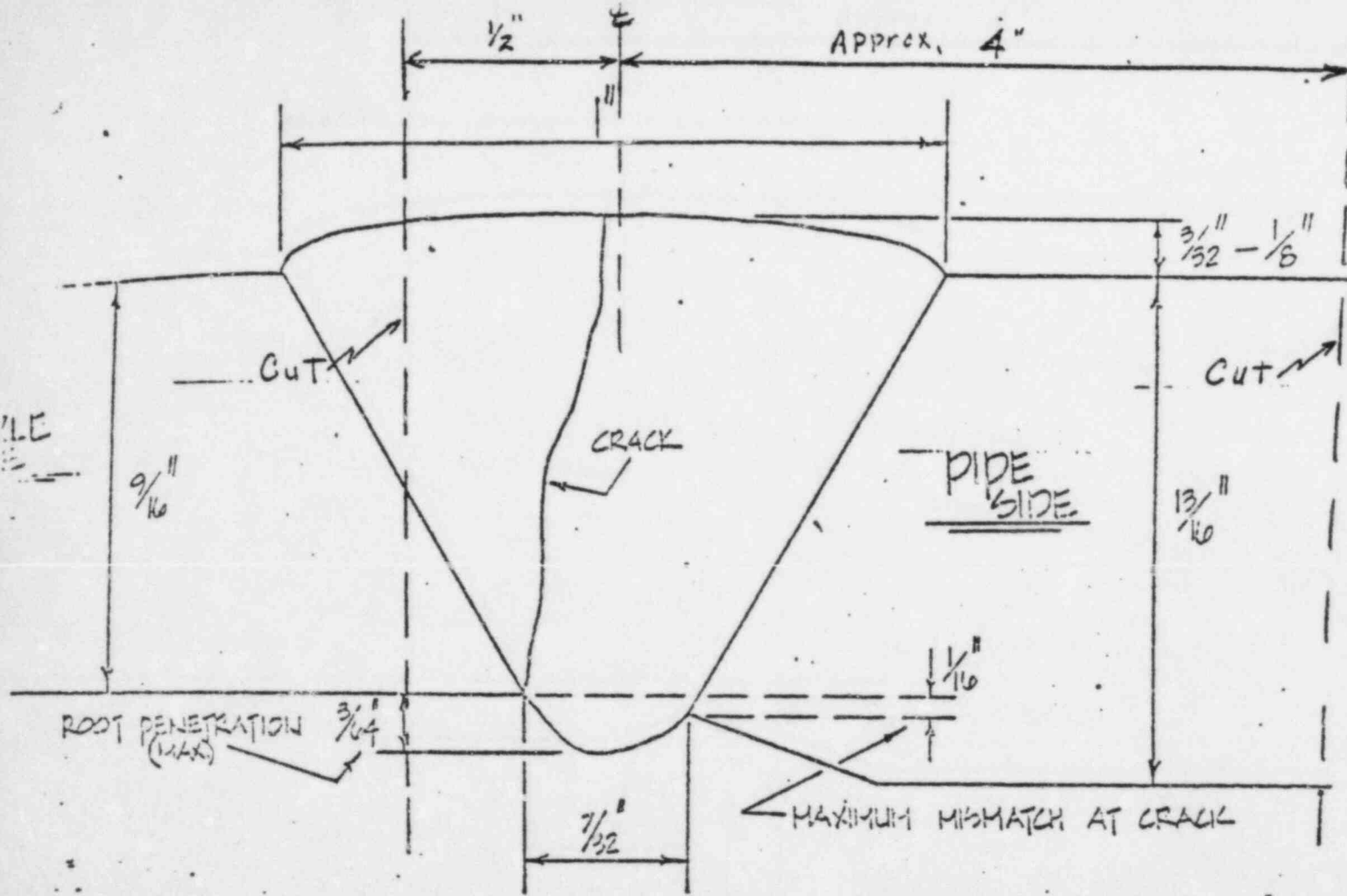
There was, however, an indication noted in F.W. 197 on generator 1-1. The indication was present on the original radiographs but had not been noted on the radiographs report. A second set of radiographs were on file for F.W. 197. This set also exhibited the same indication, however, the radiographic report did reflect the condition as being a drop thru. The indication had apparently been evaluated and determined acceptable at the time.

Because of the problem with F.W. 212 it was determined that any questionable situation should be resolved. It was therefore determined that the drop thru should be removed. A D.R. was initiated and the repair was made. The indication was removed by grinding. No welding was required. (See D.R. 3370 Attached)

#### SUMMARY

To date the metallurgical analysis has not been completed of the crack sample. We had requested, from P.G. & E., a piece of the defective material for our own analysis. This was received on Thursday, April 7, 1977 and forwarded to E. F. Gerwin in Williamsport.

It is my belief that the crack was peculiar to F.W. 212 only and not of a generic nature. Therefore, at this time we are assuming that no further repair will be required and that when the disposition of D.R. 3370 is completed the subject will be closed.



NOTE:

1. WALL THICKNESS MEASR. ARE APPROXIMATE
2. REF. TO DR. 3366 REV. A
3. STEAM GENERATOR 1-2  
F.W. #212  
V.D. 500146  
LINE # K10-555-10 IV

DR 3366 Rev. J  
ATTACHMENT # 2



# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

REWRITTEN 3-23-77

## DISCREPANCY REPORT

D.R. NO. 3366 Rev. II  
ISO. NO. 500146  
UNIT NO. 1  
CODE NO. N/A

Pacific Gas & Electric  
Diablo Canyon

SPEC. NO. 8711  
JOB NO. 7177

DATE: 3/21/77  
INSPECTOR: J. P. Runyan

DISCREPANT ITEM: F.W. #212, Line K16-555-16-IV, Steam Generator 1-2, Nozzle to pipe

### EXPLANATION OF DISCREPANCY:

A leak was detected by P.G.&E. in F.W. #212 (Pipe--Steam Gen. 1-2 Nozzle).  
Leak occurred during Hot Functional Testing March 17, 1977. Line pressure was approximately 90psi, temperature approximately 330°F.

See Attachments 1, 2 and 3.

### RECOMMENDED DISPOSITION:

- A. Cut out weld and reweld.
- B. Prior to weld cut-out:
  1. Map weld area with U.T. to determine size and location of indication.
  2. X-ray for an analysis.
  3. Remove weld material with indication for further analysis.
  4. Cut out spool 58D (F.W. #503 to 212).
- C. Reweld procedure: (Rewritten)
  1. Weld F.W. #503 using procedure 4/5 and 88/89
  2. Weld F.W. #212 using procedure 200 or 204 (See Attached Sheet #2 for additional requirements).
  3. Install soc-o-let on new spool using F.W. #411 so vent can be reinstalled.

Approved By: M.W.K. Field Q.A. Mgr. [Signature] Date 3/13/77 Customer JPL HUR M-3369 Date 3-18-77

FINAL DISPOSITION: ☐ In Accordance With Above

☒ Other (explanation and approval required)

Work Completed Insp: [Signature] Date: 3/31/77

Work Completed Insp: [Signature] Date: 3/31/77

### EXPLANATION (IF NECESSARY):

Revision I - D.R. rewritten for clarity.

Revision II See page 2

M.W.K. Field Q.A. Manager [Signature] Date 3/29/77 Customer JPL HUR M-3369 Date 3-29-77

M.W.K. Field Q.A. Manager [Signature] Date 3/23/77 Customer JPL HUR M-3369 Date 3-23-77

STEPS TO PREVENT RECURRENCE ☐ Not Applicable

- 1) Maintain preheat as indicated on Sheet 2.
- 2) Perform Post Weld Heat Treat immediately following completion of weld.

Field Q.A. Manager [Signature]

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector

ATTACH SKETCH IF NECESSARY



# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

REWRITTEN

Rev II

D.R. NO. 3366 Rev. I

ISO. NO. 500146

UNIT NO. 1

CODE NO. N/A

## DISCREPANCY REPORT

Pacific Gas & Electric

SPEC. NO. 8711

DATE: 3/21/77

Diablo Canyon

JOB NO. 7177

INSPECTOR: J. P. Runyan

DEFECT ITEM: F.W. #212, Line K16-555-16-IV, Steam Generator 1-2, Nozzle to pipe

### Recommended Disposition:

4. Drill and tap X-ray port in new spool. Install X-ray plug using seal weld F.W. #298.
5. Add all information to Iso and Process Sheets.

### Additional Requirements for F.W. #212 (See Paragraph C-2):

1. Preheat to 250°F minimum.
2. Maintain interpass at 250°F minimum, 500°F maximum until weld is complete and post weld heat treatment begins.
3. H.T. root pass; ESD-250.
4. R.T. after root and 2 layers weld added ESD-207. Repair if required and re-R.T. before completing weld.
5. Post weld heat treat at 1100°F minimum, 1150° maximum hold for 1 hour minimum, AT temperature.
6. Following post weld heat treatment, perform R.T.; ESD-207 and H.T.; ESD-209.
7. P.G.&E. will perform U.T. of final weld and maintain records.

△ 8.

### Revision II

Change para B, sub I to read "U.T. of weld performed by P.G.& E. personnel".

Add line 8 to additional Requirements: "M.W.Kellogg will perform U.T. of final weld and incorporate results into Iso package".

5/14/77  
3/20/77

Cut out & rewelded

new Prof

new Subst FW 411 - on new Prof

Cut out & rewelded

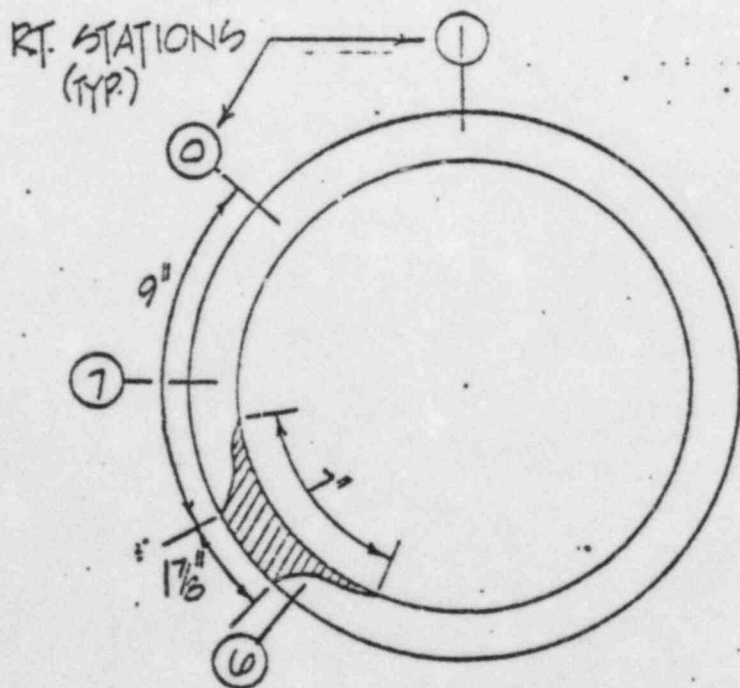
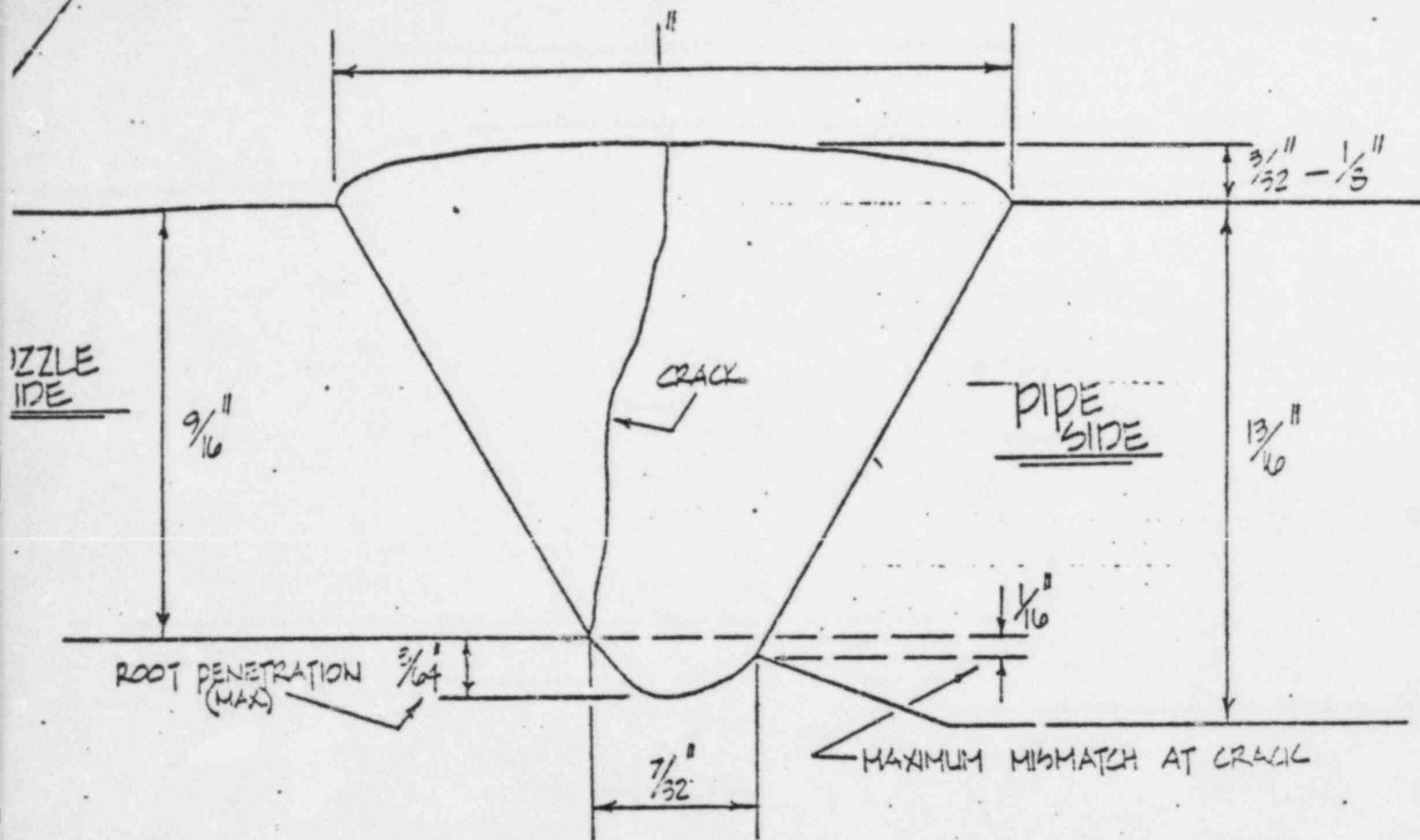
DR 3366 ROW I

Attachment #1

DWG 500146

FIELD INSTALLATION INSTRUCTIONS

NO.	DESCRIPTION	DATE	BY
1	204-201, 102	7/1	200-200
2	202	7/1	200-200
3	203	7/1	200-200
4	204	7/1	200-200



NOTE:

1. WALL THICKNESS MEASR. ARE APPROXIMATE
2. REF. TO DR. 3366 REV. A
3. STEAM GENERATOR 1-2  
F.W. #212  
NO. 500141  
-LINE # K10-553-10 IV

DR 3366 Rev I  
ATTACHMENT # 2

FOR RE-WEED  
OF FW 212

PIDE (NEW)

D/R 3366

142611-10-10

\* ID OF DIFF. MACHINES TO FIT NOZZLES

DD 3366 REV



## THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

## DISCREPANCY REPORT

D.R. NO. 3370 Rev I  
 ISO. NO. 500146  
 UNIT NO. 1  
 CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 3/23/77  
 PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: J. P. Runyan

DISCREPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle to pipe

## EXPLANATION OF DISCREPANCY:

As the result of a crack in F.W. #212 at the Feedwater nozzle on Steam Generator 1-2 (Ref. D.R. 3366), a review of the radiographs of the same welds on the remaining three (3) generators was performed by the N.R.C. In addition, the nozzle to pipe welds on the main steam leads were reviewed;

The review revealed a questionable indication in F.W. #197 on the feedwater lead to Steam Generator 1-1. The indication appears to be a burn through which contains linearly oriented voids. The accompanying radiographic report accepted the weld but did not indicate the presence of the burn through. Further investigation revealed that a second set of radiographs, shot at a later date, exist. This set also shows the burn through which was recorded on the radiographic report as acceptable (see copies attached).

(Continued on Page 2)

## RECOMMENDED DISPOSITION:

See Page 2

Approved By: M.W.K. Field Q.A. Mgr. *[Signature]* Date 3/23/77 Customer R.D. Etler Date 3/23/77 REF. PGE DR 286

FINAL DISPOSITION: ☐ In Accordance With Above☐ Other (explanation and approval required)

Work Completed Insp: \_\_\_\_\_ Date: \_\_\_\_\_

Work Completed Insp: \_\_\_\_\_ Date: \_\_\_\_\_

## EXPLANATION (IF NECESSARY):

Revision I See page 3

M.W.K. Field Q.A. Manager *[Signature]* Date 3-30-77 Customer R.D. Etler Date 3/31/77

STEPS TO PREVENT RECURRENCE ☐ Not Applicable

To be provided after further review.

See Page 3

Field Q.A. Manager *[Signature]*

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector



## THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

DISCREPANCY REPORT
 D.R. NO. 3370 Rev I  
 ISO. NO. 500146  
 UNIT NO. 1  
 CODE NO. N/A

 TOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 3/23/77  
 JECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: J. P. Runyan *JP*

 REPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle to pipe
Explanation Of Discrepancy:

To determine the severity of the indications in the burn through, we have made additional radiographs of the burn through area. These radiographs indicate that the voids are probably contained within the burn through and do not extend into the weld. There is no indication of propagation of the indications.

U.T. of the weld by P.G.&E. does not indicate any indication within the weld which is outside the acceptance criteria.

Attachments 1, 2 & 3

Recommended Disposition:

To eliminate the possibility of a potential problem at a later date, it is recommended that the burn through be removed.

The following procedure is to be followed for removal and reinspection:

1. Weld a 2" threaded coupling to the pipe as shown on the attached sketch using weld procedure 88/89 or 92/93. (FW # 827).
2. M.T. per ESD-209.
3. Post weld heat treat the weld - ESD-218.
4. Perform a hydrotest on the weld prior to drilling into the pipe.
5. Following satisfactory completion of the hydrotest, cut the hole through the pipe wall with a hole saw (Ref. ESD-230) NOTE: Additional reinforcement of 2" opening is not required per P.G. & E. calculations
6. Using an extended shaft grinder with a cone shaped rock, grind the burn through flush with the pipe surface.
7. Radiograph the area to assure complete removal of unacceptable indications.
8. Install threaded plug and seal weld (F.W. #828).

This installation is to be made in accordance with the National Board Inspection Code and ASME Section I

Reviewed by A.N.I. R.L. Sanderson 3/24/77

 DISTRIBUTION: ☒ Master O.A. File ☒ Auth. Imp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspection

## THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

DISCREPANCY REPORT

D.R. NO. 3370  
ISO. NO. 500146  
UNIT NO. ONE  
CODE NO. N/A

STOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 3/23/77  
OBJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: J.P. Runyan

DISCREPANT ITEM: F.W. #197, Line K16-554-16-IV, Steam Generator 1-1 Nozzle ot pipe

REVISION I

The following action was taken.

Add to "Recommended Disposition" the following steps.

A. To determine if there exists an ambiguity with other welds accepted by the M.K. Kellogg film interpreter who accepted F.W. 197, the following action was taken:

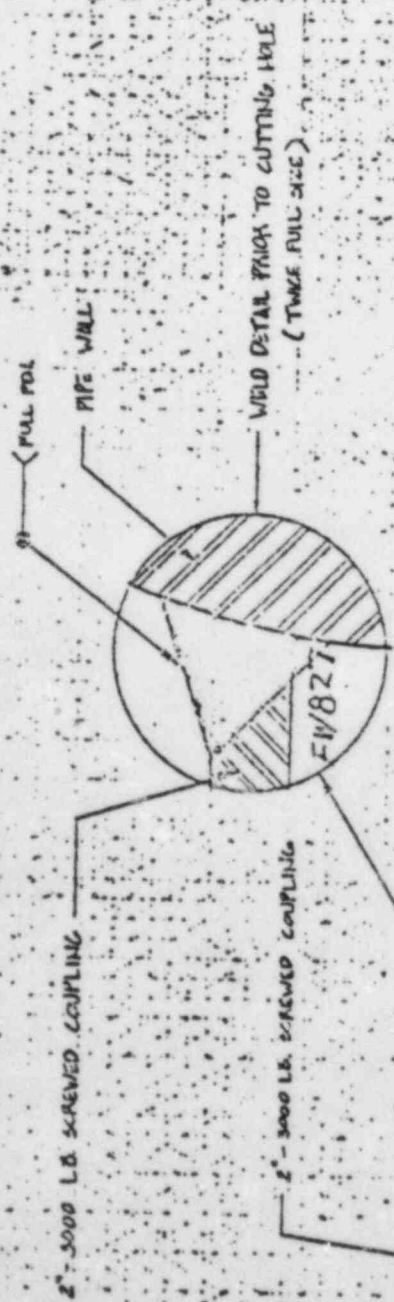
1. Review qualification records of this individual to assure compliance with SNT-TC-1A.
2. Review all radiographs which were accepted by the individual.

NOTE: The findings of the review will be reported as a separate report and attached to this D.R. before the D.R. is closed. No revision will be required to the D.R.

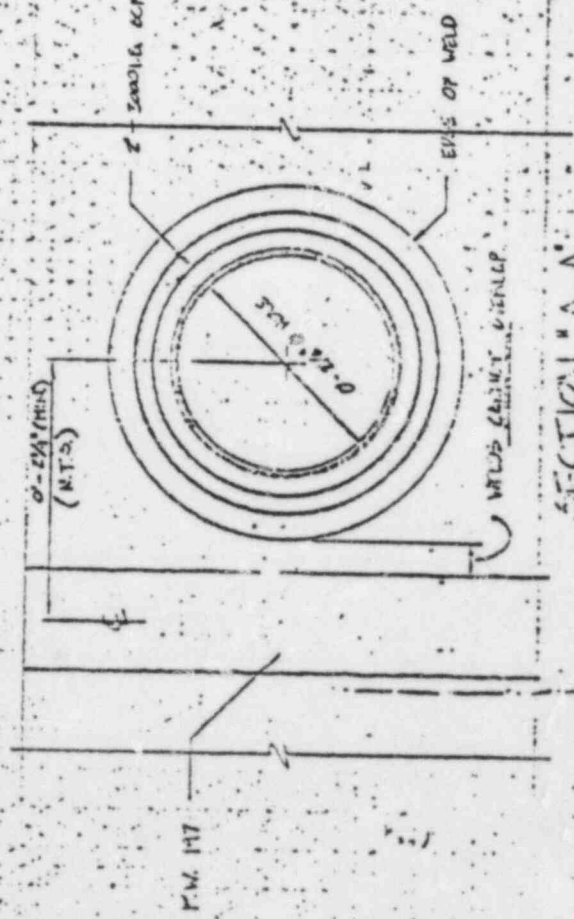
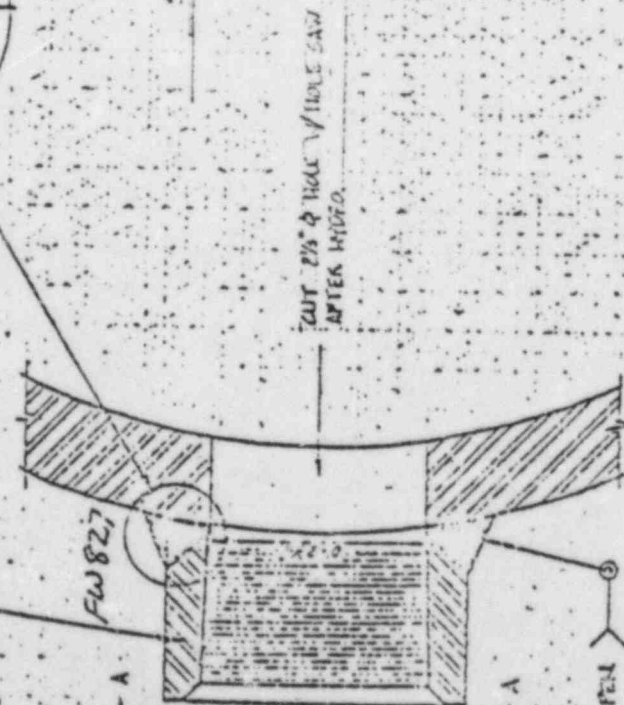
Summary:Steps to prevent recurrence:

It has been determined that no further steps are required to avoid recurrence of a similar nature. The interpretation of radiographs is always subject to be questioned after the fact. The review as required above will assure that if any problematic questions with existing installations are present, they will be corrected.

NOTE: The individual mentioned above is no longer employed by M. W. Kellogg.



SECTION THRU PIPE NEAR FW 197



SECTION "A-A"

DR 3370

WELD ON 2\"/>





DATE: 24 JUL 1954  
 1-03-500146 FEEDWATER  
 ESD-207 KRT-  
 BUTT  
 7X17  
 ASNT Level II  
 SEARS  
 HARD TANK @ 68°  
 IR 192  
 10x10  
 86 ci  
 15.5  
 16"

FITTING OR SLAB OR JOINT NUMBER	FILM NUMBER	PENETRANT	ACCEPT.	REJECT	SLAG	POROSITY	CRACK	LACK OF PEN.	LACK FUSION	UNDERCUT	SURFACE	SPRING	NOT TEAR	SAND	CRACKS	EXPOSED DATE	REMARKS	FILM INTERPRETATION BY
197	0-1-2	17/25	X													2/10/75	Depth	M. Shore
198	2-3-4		X														Two good w/c	ASNT II
199	4-5-6		X															2/11/75
200	6-7-0		X															
201																		
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APPROVED BY: [Signature]  
 ASNT II  
 2/11/75  
 2-11-75

AR 3370



# DISCREPANCY REPORT

FORM NO. 100-100-100

TO: ALCOTONE FROM: DEPT

Page 1 of 1

1. /Comp. Rollman Rollers Cont./Spec. I.D. No. U.R. 3370

FORM No. 225

## INSPECTION SECTION

Discrepant Item: Unit 1, System 3, Steam Generator 1-1 Feedwater Control Field Weld  
Id. 197.

Explanation of Discrepancy: Radiography Series revealed "drop through" with  
extensive variations. This was found after the radiograph and weld had  
been final accepted. For further details, see attached Rollman Rollers U.R. No. 3370.

[Signature]  
Field Eng./Inspector

3-23-77  
Date

## CLASSIFICATION & DISPOSITION

Minor Variation ☐

Deviation ☒

Disposition: Reject. Repair in field in accordance with P-R U.R. 3370.

The following members of the Material Review Board concur with the above Disposition:  
(For Minor Variations, only Resident Engineer's signature required.)

[Signature]  
Date

3/24/77  
Date

R. D. [Signature]  
Resident Engineer

3-23-77  
Date

[Signature]  
Date

3/24/77  
Date

[Signature]  
Responsible Engineer

3/24/77  
Date

[Signature]  
Supervising Engineer

3/24/77  
Date

## DISPOSITION ACCOMPLISHED

Remarks:

Field Engineer/Inspector

Date

## REPT TO PREVENT RECURRENCE (Deviation Only)

Engineer/Inspector

Date

Spec

Attachments

Report Completed

① JK EC MZT - H+Offine

② EISB

April 15, 1977

REC'D 4/15/77  
AT 1700 HRS

Mr. R. H. Engelken, Director  
Office of Inspection and Enforcement  
Region V  
U.S. Nuclear Regulatory Commission  
1990 N. California Boulevard  
Suite 202  
Walnut Creek, California 94596

EXHIBIT 5 TO  
ATTACHMENT 1

Docket No. 50-275  
Unit I  
Diablo Canyon Site

Dear Mr. Engelken:

On March 18, 1977 we notified the Office of Inspection and Enforcement, Region V, of a defect in a pipe discovered during testing at Diablo Canyon, which is reportable under Paragraph (e) (iii) of 10 CFR 50.55. This letter is a preliminary written report of the situation:

On March 17, 1977, during heatup for hot functional testing, a small leak was discovered in the weld joining steam generator 1-2 feedwater nozzle to the feedwater pipe. Testing was stopped to investigate the cause and to repair the leak.

The area of the leak was examined by nondestructive methods which revealed that there was a through-the-wall crack and that it was confined to the leakage area. A 16" long piece of pipe, which included that portion of the steam generator nozzle weld containing the crack, was removed from the line. A 4 1/2" long section containing the crack was cut from this pipe and delivered to PGandE's Department of Engineering Research to be analyzed to determine the failure mechanism. The investigation is still in progress.

A comprehensive review of the weld history has been performed including the weld procedure, welder qualifications, heat treatment

records, material certifications, nondestructive examinations, and water chemistry associated with the functional testing of this equipment. The results of these investigations are still under analysis.

Repair of the feedwater pipe was made by replacing a 16" length of pipe having the same specification and physical properties as the original piece. The original weld procedure was used with minor changes to preheat and postheat requirements.

Ultrasonic examinations conforming to ASME Section XI were performed on the four main steam and the other three feedwater to steam generator nozzle welds. These examinations revealed that there were no rejectable indications.

We will submit, as soon as possible, a final report documenting the failure mechanism and our intended corrective action if such action is determined to be necessary.

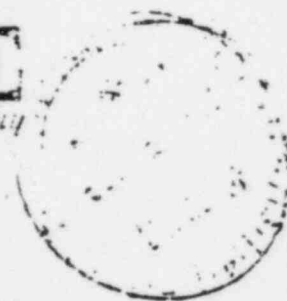
Very truly yours,

Philip A. Crane, Jr.

PAC/

cc: (3) Director of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

**PG and E**  
*Pacific Gas and Electric Company*



FAILURE ANALYSIS OF CRACKED FIELD WELD NO. 212  
DIABLO CANYON UNIT 1, STEAM GENERATOR 1-2  
NOZZLE-TO-PIPE WELD

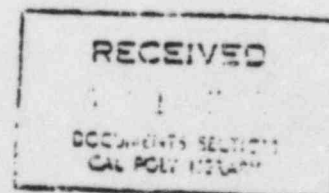
RECEIVED W/LTR DTD 9-1-77

DEPARTMENT OF ENGINEERING RESEARCH

DIABLO CANYON UNIT 1  
STEAM GENERATOR 1-2  
NOZZLE-TO-PIPE WELD

INFORMATION FILE

FOR LTR 4-1-77



# 772570078



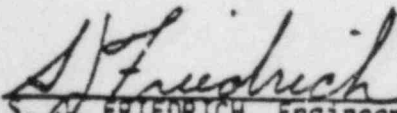
Report Issued: AUG 26 1977

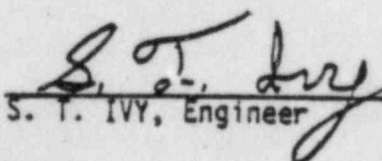
Report 411-77.55

PACIFIC GAS AND ELECTRIC COMPANY  
DEPARTMENT OF ENGINEERING RESEARCH

FAILURE ANALYSIS OF CRACKED FIELD WELD NO. 212  
DIABLO CANYON UNIT 1, STEAM GENERATOR 1-2  
NOZZLE-TO-PIPE WELD

Prepared By:

  
S. C. FRIEDRICH, Engineer

  
S. T. IVY, Engineer

Approved By:

  
W. C. HAM, Sr. Metallurgical Engr.

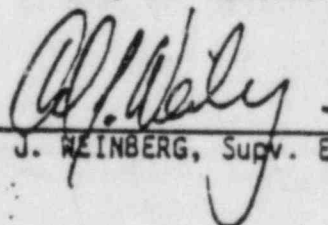
  
C. J. WEINBERG, Supv. Engineer



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## SCOPE

This report covers the metallurgical investigation of a crack found in the pipe-to-nozzle weld of feedwater line of Steam Generator 1-2 at Diablo Canyon Power Plant. Based on the findings of this investigation, recommendations are made for modification of the welding procedures, fabrication practices, and inspection of future welds made in P-3 nozzle materials.

## INTRODUCTION

On March 17, 1977, during heat-up for hot functional testing of Unit 1, a leak was discovered in the nozzle-to-pipe butt weld joining the 16-inch diameter feedwater pipe to the No. 4 nozzle of Steam Generator 1-2 (Field Weld No. 212, Line K16-555-16IV). The line temperature and pressure at this time was approximately 300°F and 90 psi, respectively. Visual observation at this time revealed an intermittent linear indication approximately 3/8-inch long running in the circumferential direction near the center of the weld. During grinding of the weld crown flush with the pipe surface the length of this linear indication increased to approximately two inches. Subsequent nondestructive examination by radiography and ultrasonics revealed a crack with a total length of approximately six inches.

As a result of the above findings, a spool piece containing a major portion of the weld, a small section of the nozzle I.D. adjacent to the weld root, and several inches of pipe were removed for analysis. Figure 1 is a sketch of the spool piece received for analysis; included is the crack path and a sketch of the shape of the crack.

## FABRICATION AND SERVICE HISTORY

### Material Specifications

The steam generator nozzle is fabricated of ASME SA508 Class 2 material. A copy of the original Material Test Report and a chemical analysis performed on the nozzle material adjacent to the crack are included in Appendix I. The chemical and physical properties for this material are within the limits for SA508 Class 2 material.

The feedwater line piping is fabricated from ASTM A106 Grade B material. A copy of the chemical analysis performed on this material adjacent to the crack are included in Appendix I. This analysis shows that the pipe material meets the chemical requirements of ASTM A106 Grade B.

Welding was performed using E8018 C-3 electrode. A copy of the original Certificate of Analysis for the three electrode lots used and a check analysis taken adjacent to the crack are included in Appendix I. These analyses show the material to within the chemical requirements of E8018 C-3.

### Welding and Heat Treating History

Welding on Field Weld 212 was done in accordance with M. W. Kellogg Procedure 200. The weld traveller and other Q.C. documents were checked along with the temperature records for preheating and postweld heat treatment for compliance with this procedure; no deviations were noted. The weld root was placed using the tungsten inert gas method (TIG) with a consumable insert and inert gas backing. Two TIG fill passes were put in and the weld was completed using shielded metal arc weld. The weld



preparation was preheated to 200°F prior to welding. Welding on FW 212 was done over a time period of several weeks; Table 1 is an outline of this welding history.

Postweld heat treatment was performed at 1150°F; metal temperatures were monitored on both the pipe and nozzle sides of the weld with a thermocouple placed several inches from the centerline of the weld. The temperature records were reviewed, and during heat-up there was an uneven heating rate between the pipe and the nozzle, resulting in a maximum temperature differential of 260°F, where the pipe was at a temperature of 680°F when the nozzle was at 420°F. This temperature differential was rectified and no further problems were encountered throughout the postweld heat treatment.

Radiography of this weld was performed after completion of welding but prior to postweld heat treatment, an acceptable practice under the governing construction code, USAS 831.1.0-1967. Review of the radiographs showed a slight density gradient in the region where the crack occurred, but it was not judged to be rejectable nor would it be interpreted as such now. If this indication in actuality is a crack, it is at the limit of detectability.

#### Service History

Prior to making FW 212, a cap was welded onto the feedwater nozzle of SG 1-2, and a hydro test of the steam generator was performed. Subsequently, the piping was attached and two more hydro test cycles were performed, followed by a hot functional test program. A second hot functional was underway when the leak occurred. This sequence of events is summarized in Table 1, the hot functional testing summary is in Appendix II.

During the period between the hydro test following weld completion, August 29, 1975, and the time the leak occurred, March 17, 1977, the system was filled with water under chemical control to limit corrosion or other damage to the piping or steam generator internals. The water chemistry records were reviewed and no abnormal conditions were noted.

#### RESULTS OF METALLURGICAL EXAMINATION

The surface of the main fracture was almost flat, originated along the edge of a weld bead, grew approximately perpendicular to the surface and contained markings that ran parallel to the original edge, Figures 2 and 3. These markings appeared to be "beachmarks" denoting positions of the crack during its growth. The fracture surface appeared to be oxidized and was dark in color near the origin edge. Using SEM, the main fracture surface was found to be oxidized, to exhibit some evidence of mechanical damage (rubbing) and, at a distance from the origin, to contain secondary cracks, Figure 4. The presence of the visual beachmarks and the secondary cracks indicate that cyclic loading had a role in the generation of the fracture.

Examination of several polished cross-sections taken through the crack showed the fracture initiated in small regions of lack of fusion along the fusion zone and in small, up to 0.015-inch deep, cracks at the root of grinding scratches adjacent to the fusion zone on the nozzle side of the weld. The crack propagated through the heat-affected zone and then through the weld metal in the V-shaped section of the weld, Figure 5. The crack cross-section in the heat-affected zone was very flat and straight, indicating little ductility; once into the weld the crack tended to branch

and wander, indicating a material with more ductility. The heat-affected zone in both pipe and nozzle materials was tempered martensite, the pipe material away from the weld was mixed pearlite and ferrite, and the weld material was tempered martensite. Microhardness tests in these various regions, converted to Rockwell hardness values, were as follows: nozzle, away from HAZ,  $R_B 96$ ; nozzle HAZ,  $R_C 34$ ; weld root,  $R_C 23$ ; pipe base material,  $R_B 88$ .

Examination of the small cracks adjacent to the main fracture, Figures 6, 7, and 8, illustrated that the cracks originate in slight depressions in the surface, grew perpendicular to the surface and at an oblique angle to the rolling direction (as revealed by the inclusion stringers), were filled with oxide, were transgranular, and exhibited some crack branching. No evidence of plastic deformation was observed. EDXA microprobes verified that the material in the cracks was probably oxide, that the base metal was carbon steel, and that the inclusion stringers were manganese sulfide, Figures 10-12.

Two small cracks were opened up for examination by impacting them at liquid nitrogen temperatures (Specimens 2 and 3). The fracture surfaces had a dark layer of oxide patches along the outer edge, Figure 9, and were in close proximity to weld beads. Each crack was actually the result of the coalescence of many cracks each having its own origin. The origin edges of the fracture surfaces were covered with a relatively uniform oxide layer. The oxide layer was present all the way to the tip of the prior existing crack.

Using the plastic/carbon replica technique, the details of the main fracture surface were examined in the TEM. The surface was again found to be completely oxidized; however, in some areas, the oxide formed parallel ridges which could possibly suggest prior fatigue striations. In some areas, large block-like oxide growth was found.

Extraction replicas were examined in the TEM to obtain the diffraction pattern of the oxide. The patterns obtained were analyzed, Appendix II, and found to be composed of  $\alpha\text{-Fe}_2\text{O}_3$ . This oxide forms at about 300°C (570°F).

#### CONCLUSIONS

The characteristics of the small cracks and the major fracture surface are similar in regularity, orientation, oxidation, and degree of crack branching. The oxide extended to the tip of the advancing cracks indicating that oxidation and/or corrosion was an inherent part of the crack growth process. The characteristics of the fracture process; i.e., transgranular, oxidized, no plastic deformation, with some crack branching, indicated that the crack growth mode was either corrosion fatigue or thermal fatigue.

The oxide resulting from corrosion fatigue would probably be hydrated and a lower temperature form than found on the crack surfaces in this study. Based on this and the closely regulated water purity, it is believed that it is more likely that the cracks in the feedwater line of Diablo Canyon Unit 1 steam generator resulted from thermal fatigue. The cracks initiated in elongated depressions in the surface of the steel which are believed to be grinding marks, and apparently acted as local stress risers, and at minor regions of lack of fusion in the weld root.

It is believed that small cracks initiated on the I.D. of the nozzle, weld, and pipe during the thermal cycling that occurred during preheating. These small cracks originated at convenient stress risers such as grinding scratches and regions of lack of fusion and weld bead, rollover. At the time of acceptance radiography, prior to postweld heat treatment, these cracks were below the limits of detectability for radiography. Upon heat-up for the postweld heat treatment thermal stresses from the uneven heat-up between the pipe side and nozzle side of the weld caused several of these small cracks to link up and grow to some substantial size, most likely through the heat-affected zone (HAZ) and into the weld metal. The HAZ at this time, prior to postweld heat treatment (PWHT), would have consisted of untempered martensite, and have a hardness of about  $R_c 45$ . The HAZ would have quite low ductility in this condition and easily propagate an existing crack when subjected to differential thermal stresses. Once this crack had propagated to a substantial size during PWHT it propagated to failure by fatigue as a result of subsequent hydro test and hot functional test stresses.

Since postweld heat treatment of the weld has tempered the heat-affected zone, this high hardness, low ductility condition no longer exists. The heat affected-zone now has a hardness of  $R_c 30-35$ , and would not be expected to be subject to the rapid crack growth that is believed to have occurred during PWHT.



## RECOMMENDATIONS

Based on the findings of this investigation, following recommendations are made for future welds made in P-3 materials:

1. Specify a minimum preheat temperature of 250°F; a maximum interpass temperature of 550°F should also be specified.
2. Maintain the preheat temperature throughout the weld and until postweld heat treatment can be initiated. If a long delay during welding or between welding and PWHT is unavoidable, maintain the preheat for at least eight hours after welding stops, and then slowly cool to room temperature.
3. Care should be exercised in avoiding large temperature differentials during preheating and postweld heat treatment.
4. Final radiography should be done after postweld heat treatment.

TABLE 1

## Fabrication and Service History (Summary)

1-14-73	Preheat to attach lugs
1-14-73	Preheat to weld cap
1-15-73	Preheat to weld cap
2-19-73	Hydro, 1356# at 106°F
4-8-74	Preheat to remove cap
4-9-74	Preheat to remove cap
5-18-74	Preheat for nozzle-pipe weld
5-21-74	Preheat for nozzle-pipe weld
5-22-74	Preheat for nozzle-pipe weld
5-23-74	Preheat for nozzle-pipe weld
5-24-74	Preheat for nozzle-pipe weld
6-15-74	Radiograph finished weld
6-24-74	PWHT weld
8-74	PWHT adjacent weld (15" away)
8-29-75	Second hydro, 1320# at 88°F
8-29-75	Third hydro, 1356 at 88°F
12-13-75 to 2-10-76	First hot functional
3-16-77	Second hot functional

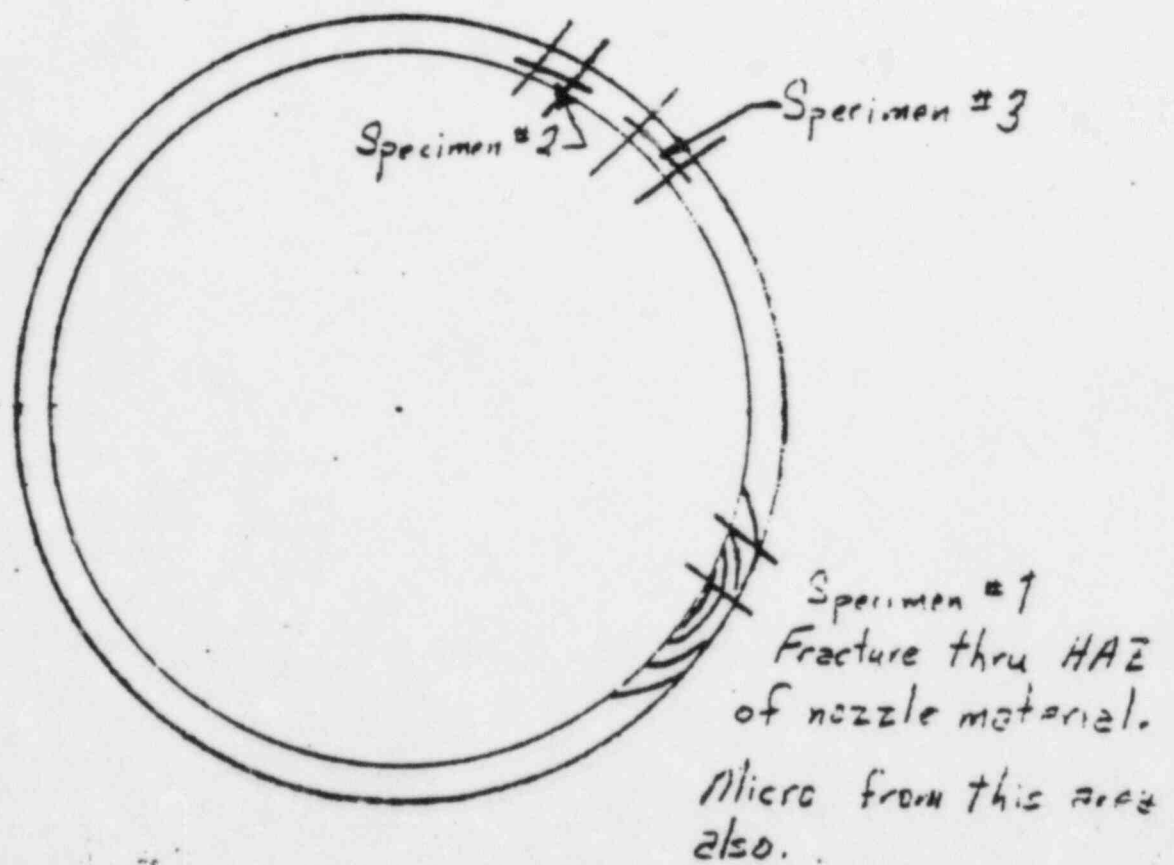
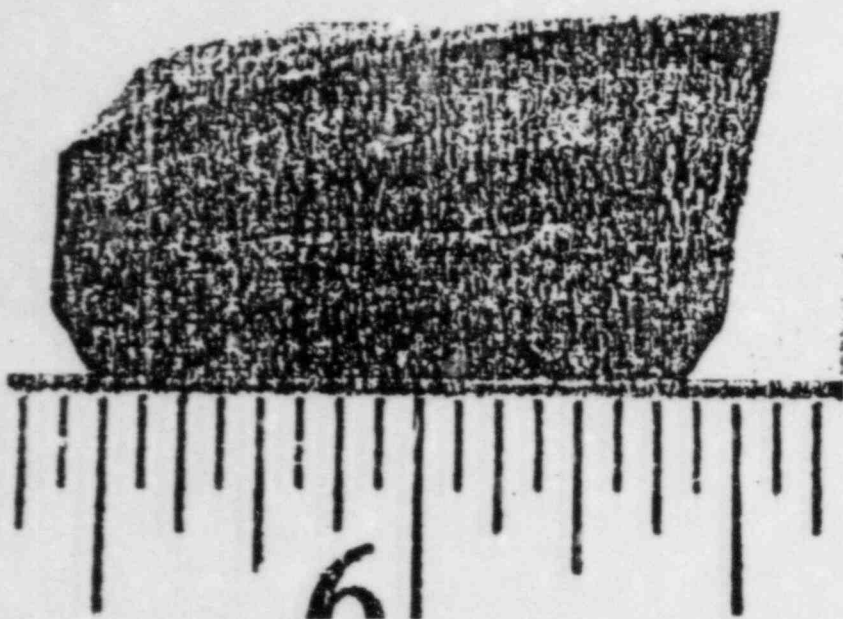


Figure 1. Schematic diagram of failed nozzle illustrating the locations from which samples were removed.

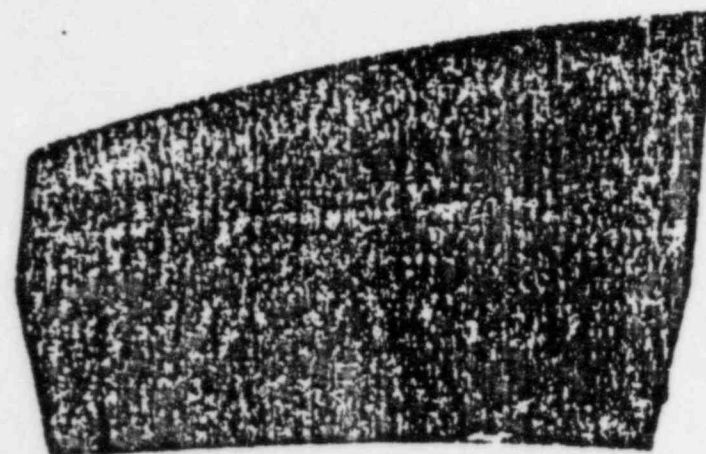


Figure 2. Overall view of fracture surface.



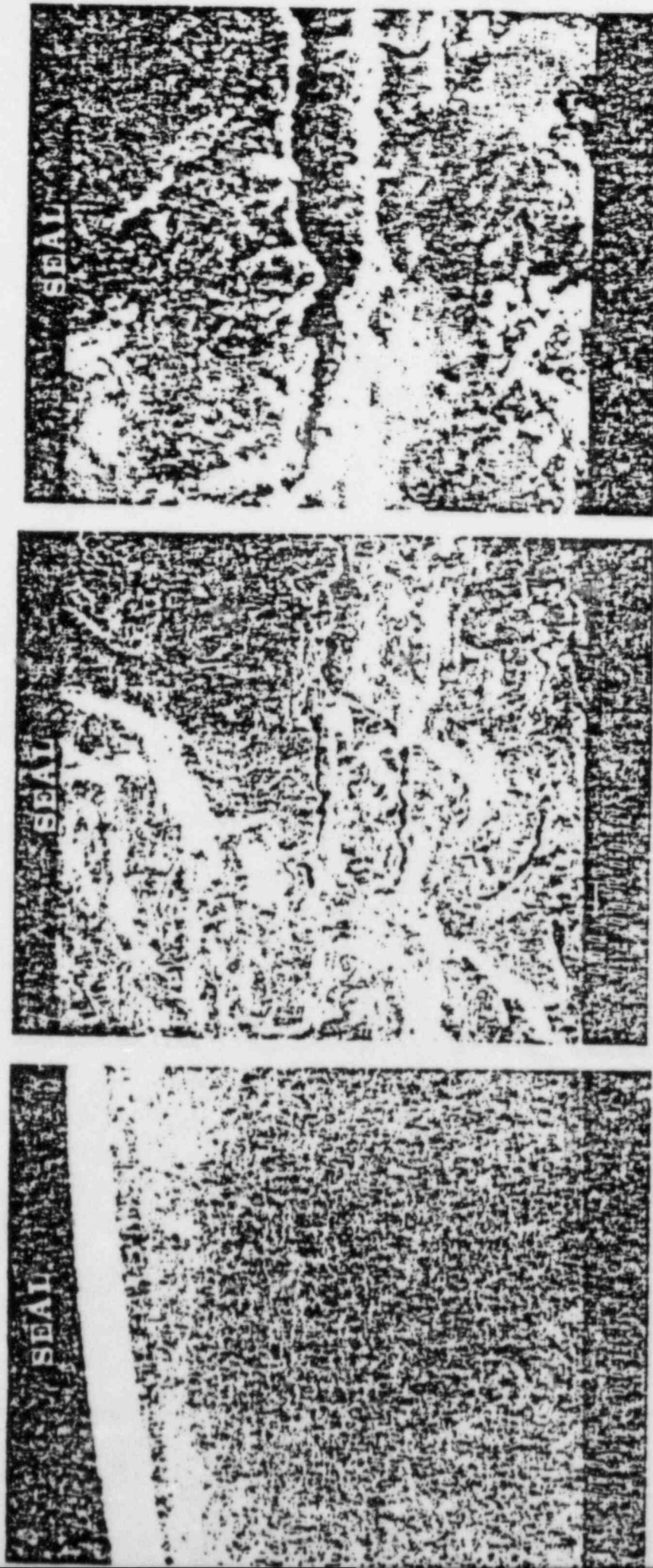


a.



b.

Figure 3 Specimen #1, showing fracture surface with parallel markings originating along one side of weld bead; 3X.



a. b. c.

Figure 4. Surface of main fracture which was in the heat affected zone (HAZ) of the nozzle, Specimen #1, showing oxidized surface with crack branching. a. 20X (50595). b. 500X (50594). c. 2000X.

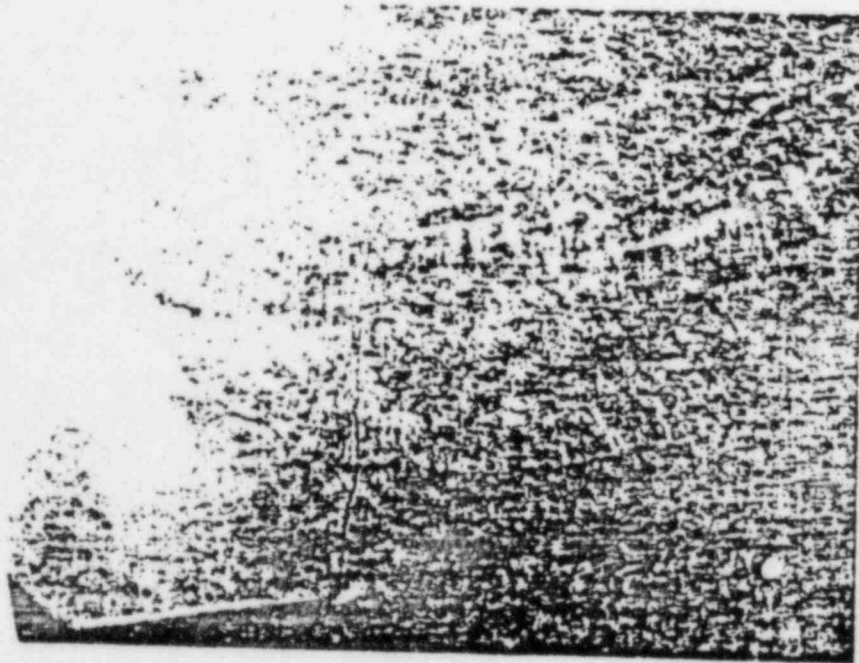
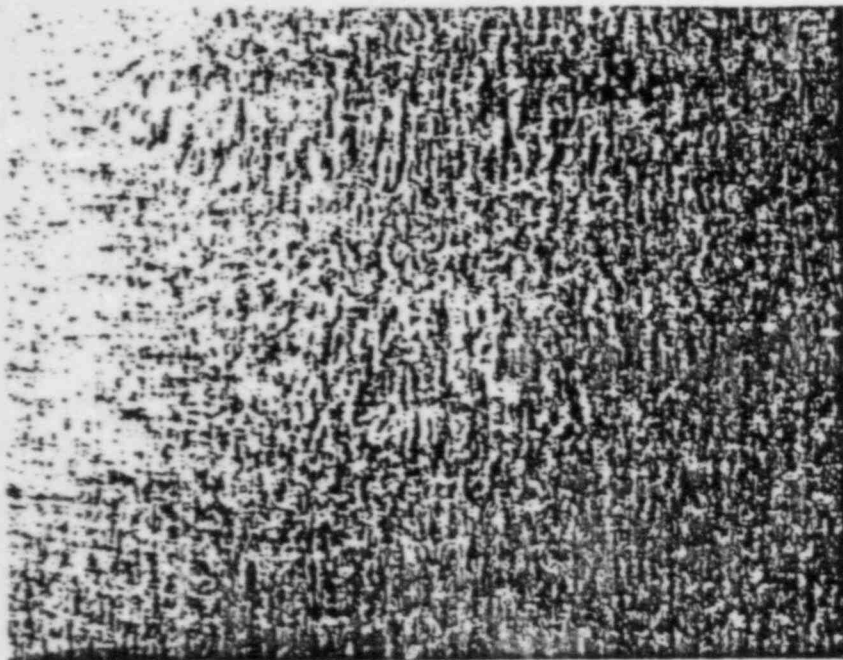


Figure 5. Polished cross section of the main fracture. The pipe is on the bottom left, the weld root is bottom center and the nozzle is bottom right. The bottom of the picture is the pipe I.D. The small bright dots are micro hardness indentations. 7 x magnification.



a.



b.

Figure 6. Cross section of Specimen #3, Figure 1, which shows small secondary cracks similar to those noted adjacent to the main fracture. The tilted view shows the coincidence of the cracks with grinding marks on the pipe and nozzle I. D., as well as in the unground weld.



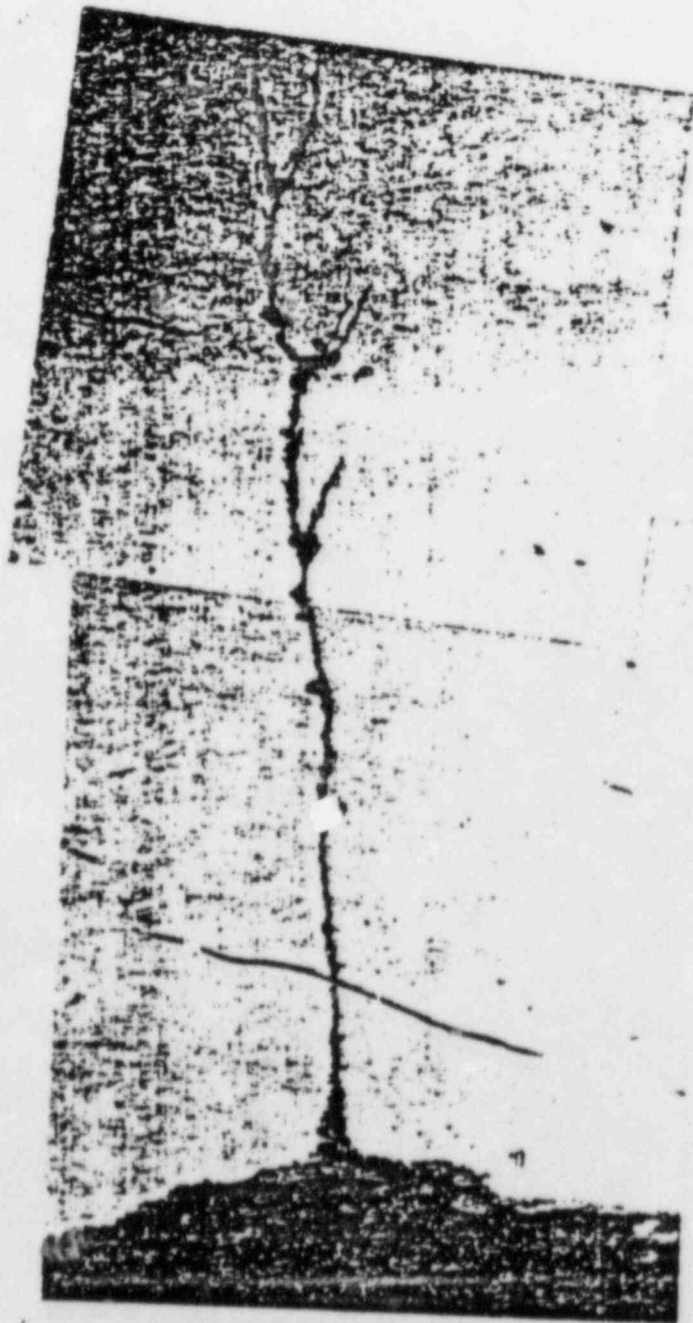
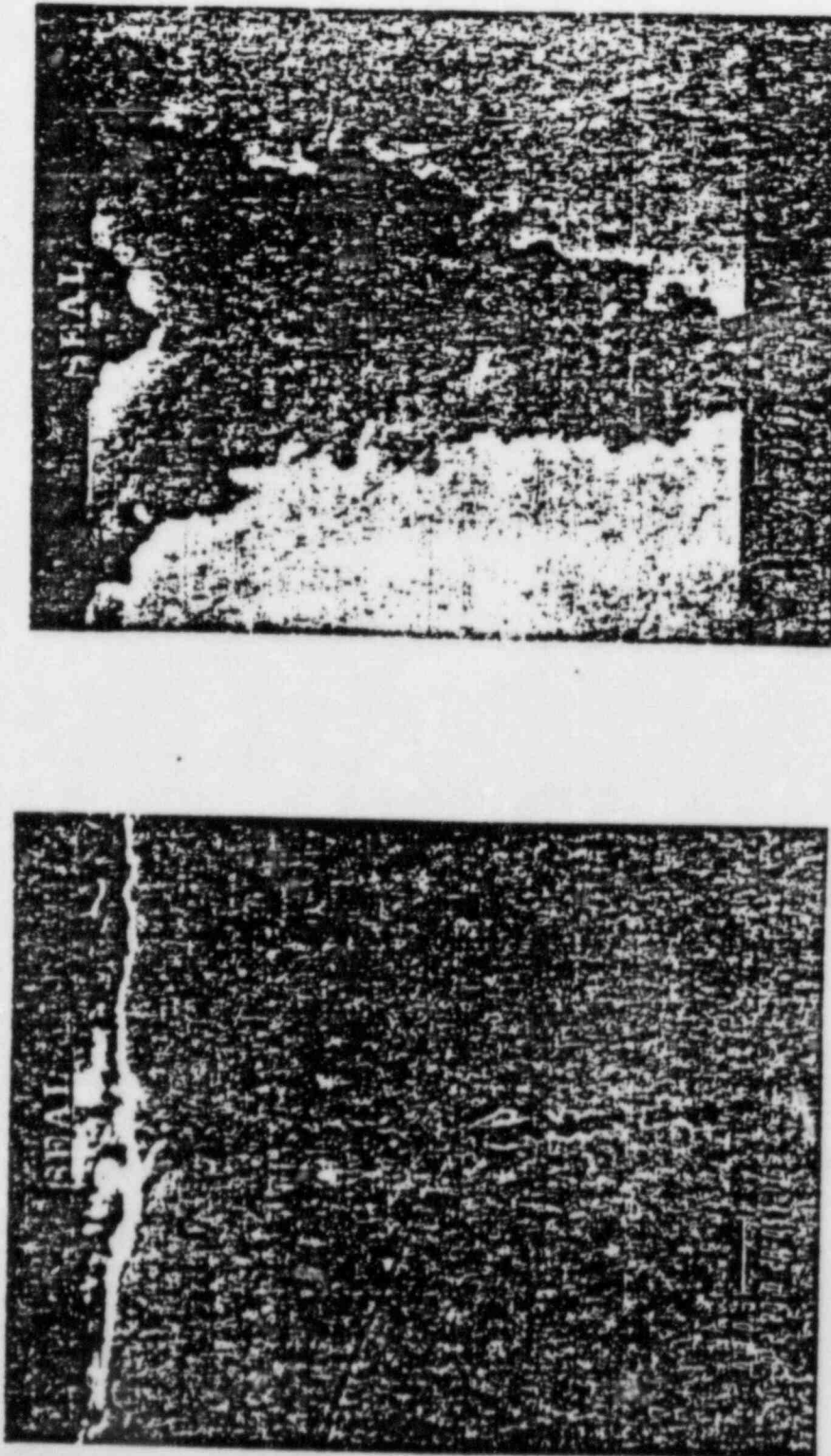


Figure 7. Polished cross section from main fracture area showing a crack, filled with oxide, with crack branching and sulfide inclusion stringers; 450X.



a. b.  
Figure 8. Details of polished cross section shown in Figure 7 a. 1000X.  
(51136). b. 5000X (51138).

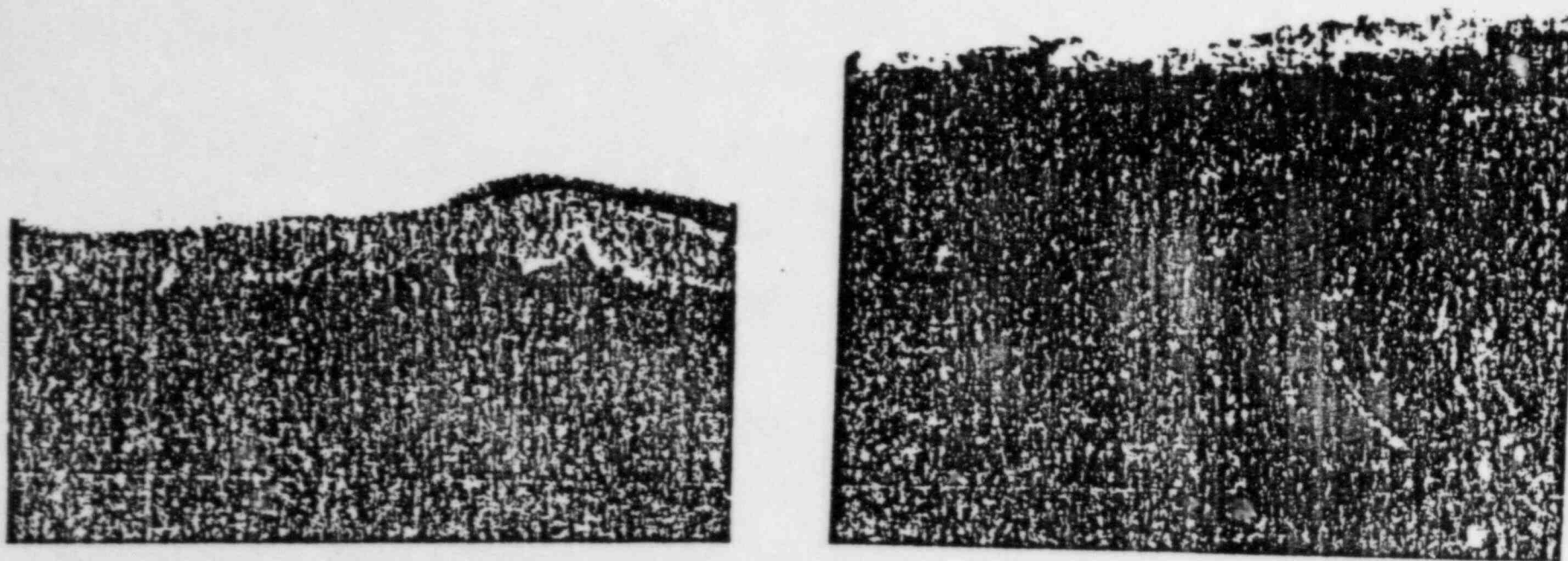


Figure 9. Opened crack along weldment on Specimen 2 showing several fracture plateaus indicative of multiple cracks and multiple origins. a. 3X. b. 12X.

Figure 10. EDXA microprobe spectrum obtained from steel matrix in Figure 8.

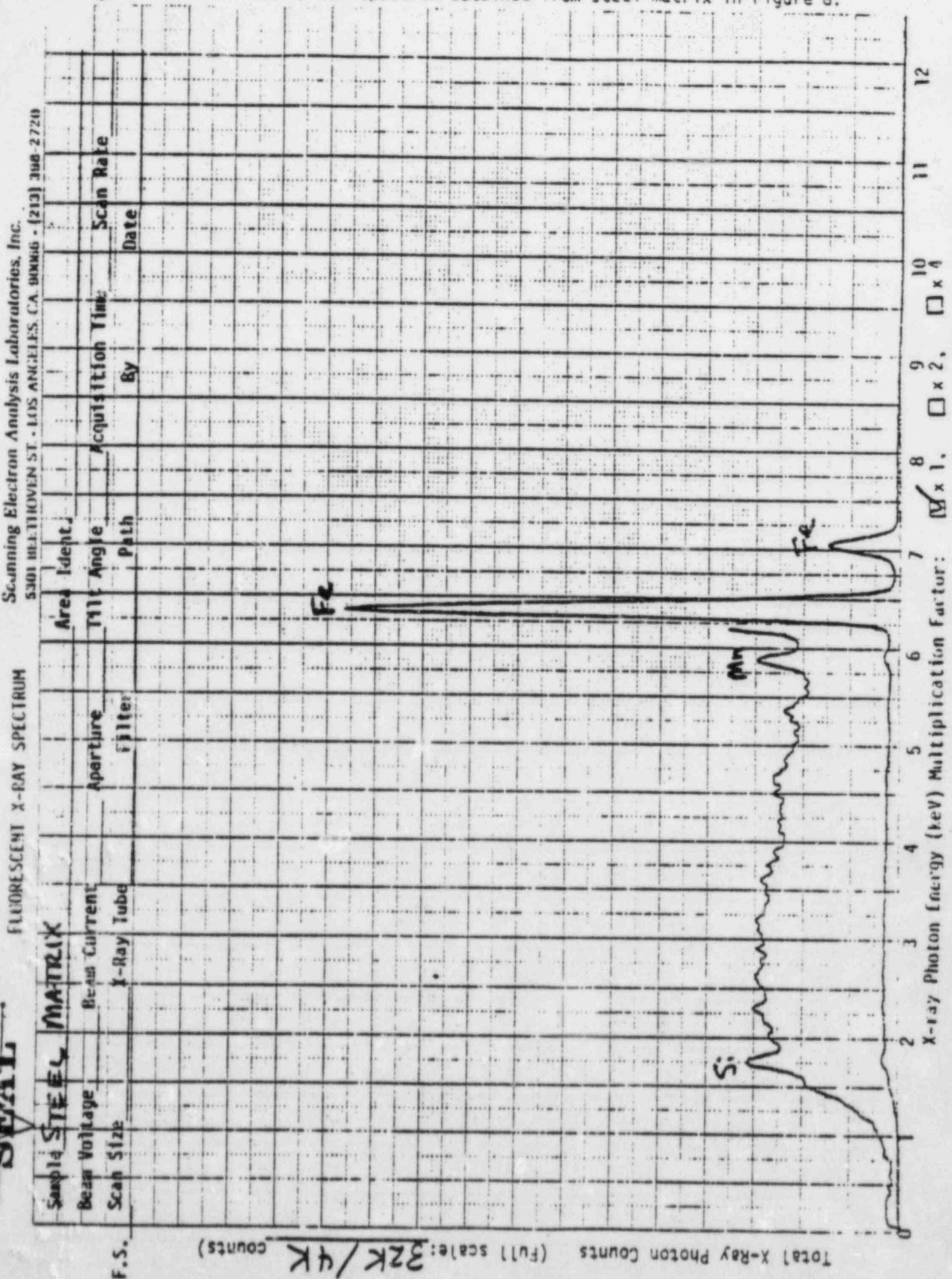




Figure 11. EDXA microprobe spectrum obtained from oxide in crack in Figure 8.

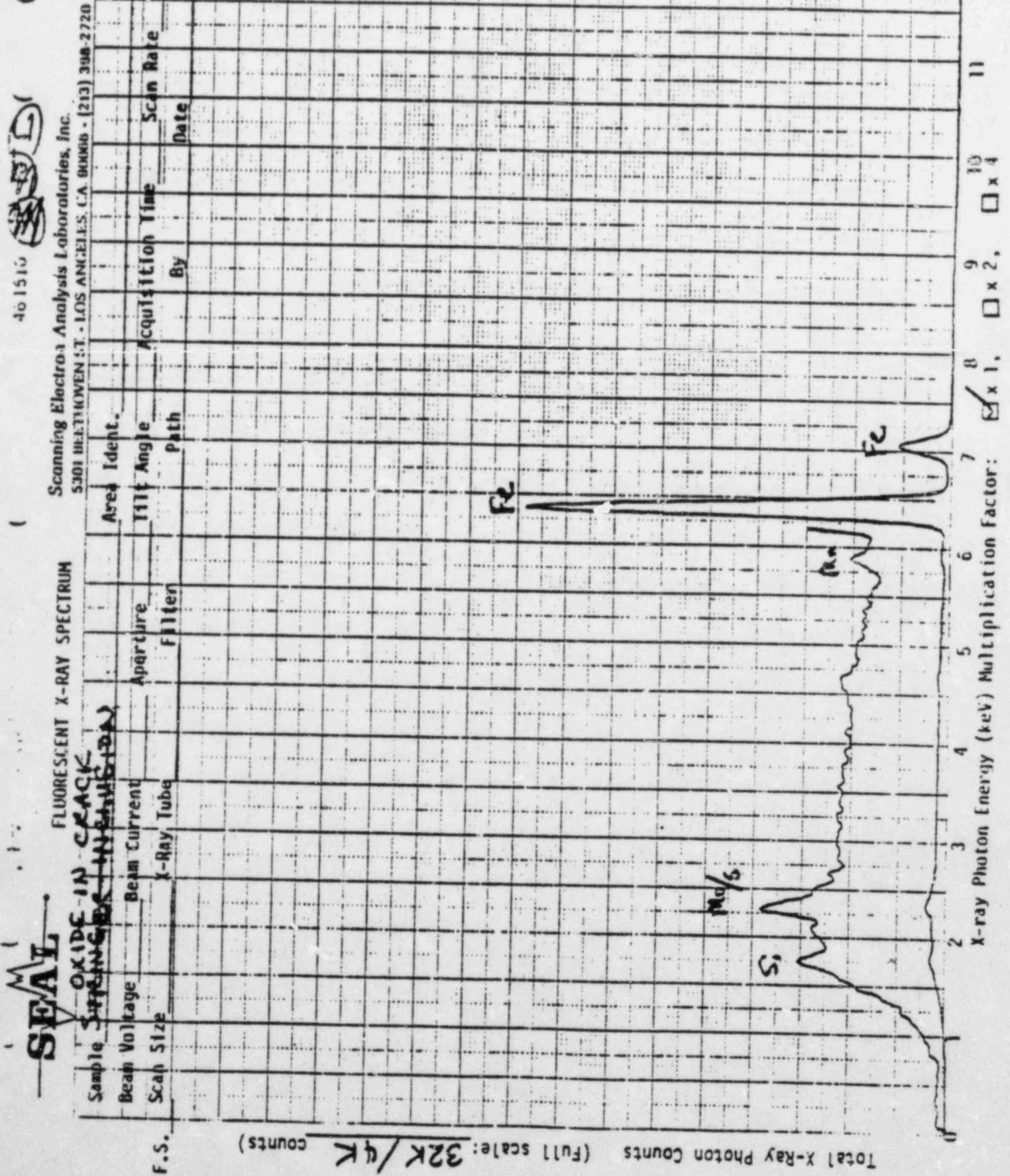
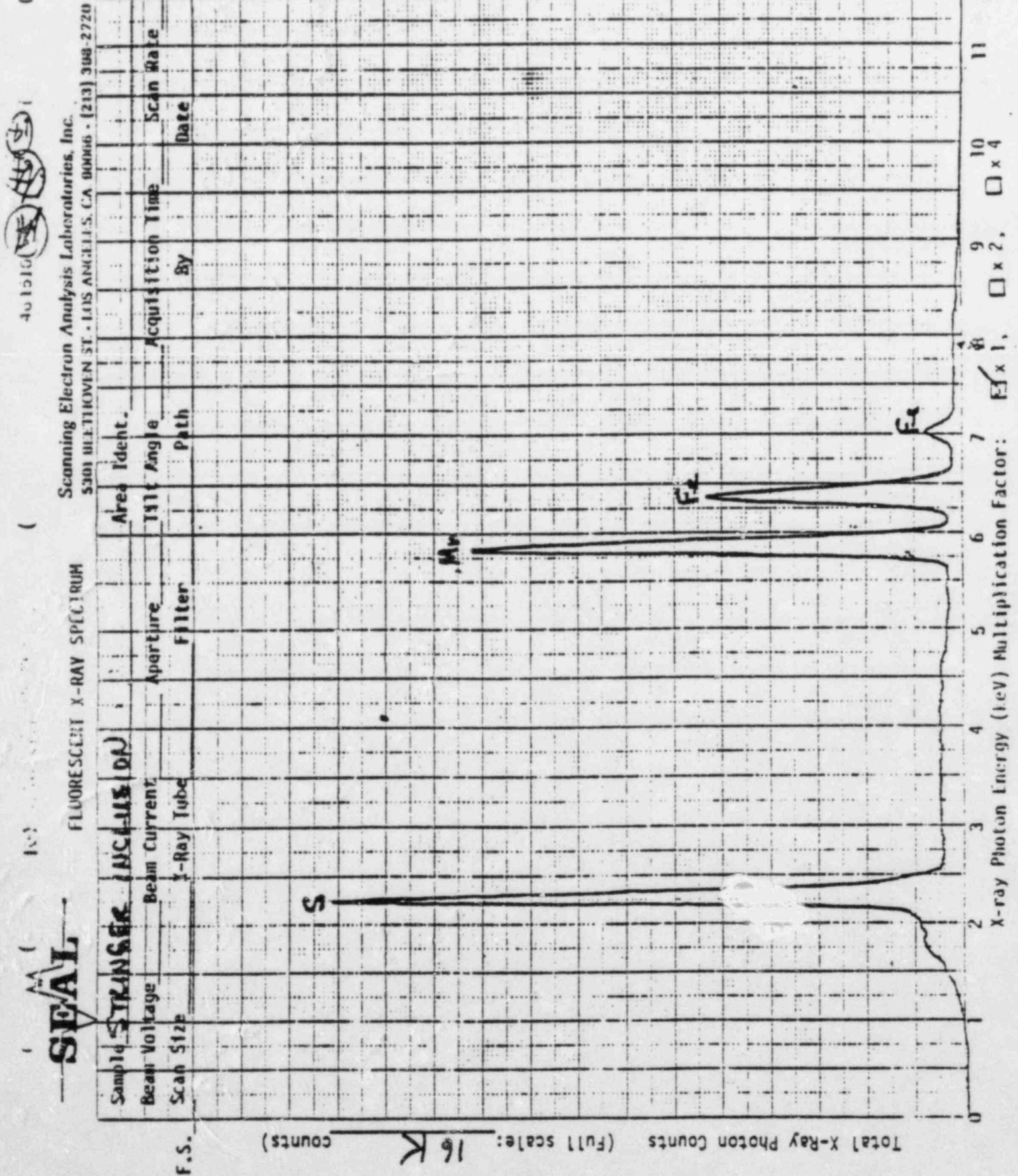




Figure 12. EDXA microprobe spectrum obtained from inclusion stringer Figure 8.



APPENDIX I

Material Specifications and Welding Procedure

# FEDERAL BUREAU OF INVESTIGATION

## MATERIAL TEST REPORT

S.O. No. \_\_\_\_\_

DATE June 20, 1969

Purchaser Westinghouse Electric Corp.

Purchaser's Order No. 54-5-1042-890

Distributor G.L. Crain Co., Inc.

Distributor's Order No. \_\_\_\_\_

QTY.	PRODUCT	SPEC.	HEAT OR CODE NO.	FORGING NO.	HEAT TREATMENT	ICE CHARTS ATTACHED	
						YES	NO
1	14.75" I.D. Inert Stud Feedwater Nozzles per/Dwg. 870G217  T-00784	SA508-2	Q2Q3W	L01R4	1650°F ± 25°F. 1hr/in Air Cooled. 1560°F ± 25°F. 1hr/in Water Quench 1290°F ± 25°F. 1hr/in Air Cooled. 1560°F ± 25°F. 1hr/in Water Quench 1290°F ± 25°F. 1hr/in Air Cooled. Tests Stress Relieved At: 1125°F ± 25°F. For 20 Hours.		X

### CHEMICAL ANALYSIS AND MECHANICAL PROPERTIES

FORGING NO.	HEAT NO.	C	MN	P	S	SI	CR	NI	MO	V		NDT REPORTS ATTACHED			REMARKS
												U.T.	M.P.	D.P.	
L01R4	Q2Q3W	.21 .209	.79 .79	.012 .005	.012 .015	.27 .29	.34 .37	.80 .80	.68 .57	.05 .04	Lidle Check	X	X		

FORGING NO.	HEAT NO.	TEST TEMP.	TENSILE PSI ± 1000	YIELD PSI ± 1000	ELONG. % in 2"	R.A. %	B.H.N.	IMPACT TESTS (Keyhole @ 10° F.)		
								ENERGY (ft.-lbs.)	LATERAL EXP. (in.)	% SHEAR
L01R4	Q2Q3W	Room Temp.	93.000	72.500	21.5	64.9		92-109-114	.067-.078-.080	80-90-90
		180°	93.100	73.400	21.5	60.9		108-101-103	.074-.076-.073	85-90-85

*Chief de la Republique*

We hereby certify the above results to be correct  
as determined by the results of the Corrosion

# CHEMETRON CORPORATION

WELDING PRODUCTS DIVISION

447

## Certificate of Analysis

M. W. Kellogg Co.  
c/o Pacific Gas & Elect. Co.  
Daiblo Canyon Power Plant  
Avila Beach, California 93424

Customer Order No. N/AOrder No. 62853Shipped 6-6-73This material conforms to Specification AWS A 5.5-69Test No. 415

X-Ray Satisfactory

F-7177-1269 Item AType E 8018 C-3

Trade Name:

Atom Arc 8018

Moisture @ 1800°F. 0.2%

Diameter Size:

1/8"Concentricity 3%1,000 lb.Type Steel A-285

Lot Number:

C315C3AD

Heat Number:

402K9921

Test No.	Full	Split	Volts	Amps
Tensile & Impacts	1	6	22	140

Carbon

.05

Manganese

1.01

Chromium

Nil

Nickel

.96

Silicon

.58

Columbium

Tantalum

Molybdenum

.12

Tungsten

Copper

Titanium

Phosphorus

.017

Sulphur

.018

Vanadium

.02

Iron

Ferrite

Test Results:	AS Welded
Yield	76,400
Tensile	87,500
Elongation	31%
Red. of Area	76.7%

Charpy V-Notch Impacts Tested @ -40°F.Impacts 79Impacts 84Impacts 87

APPROVED

M. W. KELLOGG

Q. A.

P.G.E.E. Daiblo Canyon Project

7W 6-27-73

INITIALS

State of Penna. )  
County of York ) SS

Subscribed and sworn to before me  
this 22nd day of June

19 73

The undersigned certifies that this report is correct and that no significant change has been made in any of the elements described in the qualification approval.

CHEMETRON CORPORATION  
WELDING PRODUCTS DIVISION

BY

R. J. Stallman

My commission expires:

4-12-76



# CHEMETRON CORPORATION

WELDING PRODUCTS DIVISION

520

## Certificate of Analysis

M. W. Kellogg Company  
c/o Pacific Gas & Electric  
Power Plant  
7 Miles N. of Avila Beach  
Avila Beach, California 93424  
7177-1505

Customer Order No. 6520  
Order No. 65365  
Shipped 9-21-73

This material conforms to Specification AWS A 5.5-69

Test No. 650

X-Ray Satisfactory Type E 8018-C3

Trade Name: Atom Arc 8018

Moisture @ 1800°F. 0.2%

Diameter Size: 1/8"  
400 lb.

Concentricity 3%

Type Steel A-285

Lot Number: B328C3AD  
Heat Number: 627233

Test No. Full Split Volts Amps

Tensile &  
Impacts

1 6 22 130

Test Results: AS  
Welded

Yield 72,400  
Tensile 86,000  
Elongation 31%  
Red. of Area 75.0%

Carbon .05  
Manganese .93  
Chromium .05  
Nickel 1.07  
Silicon .40  
Columbium  
Tantalum  
Molybdenum .15  
Tungsten  
Copper  
Titanium  
Phosphorus .014  
Sulphur .022  
Vanadium .02  
Iron  
Ferrite

Charpy V-Notch Impacts Tested @ -40°F.

Impacts 76  
Impacts 77  
Impacts 78

Filletts: OK Vertical 1 Overhead 1

State of Penna. )  
County of York ) SS

Subscribed and sworn to before me  
this 25th day of Sept.

1973



The undersigned certifies that this report is correct and that no significant change has been made in any of the elements described in the qualification approval.

CHEMETRON CORPORATION  
WELDING PRODUCTS DIVISION

SEAL

Notary Public

My commission expires:

4-12-76

BY



## CHEMETRON CORPORATION

WELDING PRODUCTS DIVISION

979

## Certificate of Analysis

M. W. Kellogg Co.  
c/o Pacific Gas & Electric  
Diablo Canyon Power Plant  
Mark: PO F7177-2373  
7 Miles North of  
Avila Beach, California

Customer Order No. E 7177-2373Order No. 78322Shipped 2-20-75

This material conforms to Specification AWS A 5.5-69 & M. W. Kellogg Spec.  
Test No. 558 B31.7 SFA 5.5 Class II  
X-Ray Satisfactory Type E 8018-C3

Trade Name: Atom Arc 8018Diameter Size: 3/32"  
200 lb.Lot Number: L416F3AC  
Heat Number: 640912

Moisture @ 1800°F. 0.2%  
Concentricity 4%  
Type Steel A-285

Test No. Full Split Volts Amps

Tensiles &  
Impacts 1 7 21 140

Test Results: AS Welded Stress Relieved  
8 hrs. @ 1125°F.

Yield 72,100 / 68,500 /  
Tensile 84,500 / 82,100 /  
Elongation 30% / 32% /  
Red. of Area 77.5% / 79% /

Charpy V-Notch Impacts Tested @ -20°F.

Impacts 96-109-110-113- 85-99-100-103-  
117

Lat. Exp. 73-76-83-79-77 71-84-79-78-7  
% Shear 90-85-90-85-90 75-70-80-80-7

Filletts: OK Vertical 1 Overhead 1

State of Penna. )  
County of York ) SS

Subscribed and sworn to before me  
this 25th day of Feb.



The undersigned certifies that this report is correct and that no significant change has been made in any of the elements described in the qualification approval.

SEAL

Annita B. Olmstead  
Notary Public

My commission expires: 8-21-78

CHEMETRON CORPORATION  
WELDING PRODUCTS DIVISION

BY

R. W. Boyer  
R. W. Boyer

27  
LABORATORY CERTIFICATE

Report 411-77.55

Anamet Laboratories, Inc.

ANALYTICAL  
CHEMICAL  
METALLURGICAL

HIGH TEMPERATURE

May 25, 1977

LABORATORY NUMBER:

577.329

P.O. No. LO-879806

SAMPLE:

One (1) Sample for  
Chemical Analysis

MARK:

P-12 B  
DCPP #1 Steam Generator  
Nozzle Failure Analysis

DATE SUBMITTED:

May 24, 1977

REPORT TO:

Pacific Gas & Electric Company  
3400 Crow Canyon Road  
San Ramon, California 94583

Attn:

Mr. Steve Ivy

CHEMICAL ANALYSIS

				<u>Requirements</u>	
				A-508 Cl. 2	
				<u>Min.</u>	<u>Max.</u>
Carbon	(C)	0.23%	-		0.27%
Chromium	(Cr)	0.40%	0.25%		0.45%
Manganese	(Mn)	0.81%	0.50%		0.90%
Molybdenum	(Mo)	0.58%	0.55%		0.70%
Nickel	(Ni)	0.80%	0.50%		0.90%
Phosphorus	(P)	0.009%	-		0.025%
Silicon	(Si)	0.28%	0.15%		0.35%
Sulfur	(S)	0.012%	-		0.025%
Vanadium	(V)	0.05%	-		0.05%

Respectfully submitted,

ANAMET LABORATORIES, INC.

By

Siegfried Otto  
Manager, Testing

3c  
jm

28  
LABORATORY CERTIFICATE

Report 4'1-77.55

Anamet Laboratories, Inc.

2827-79 STREET

BERKELEY, CALIFORNIA 94710

841-5771

ANALYTICAL  
CHEMICAL  
METALLURGICAL

HIGH TEMPERATURE  
APPLIED RESEARCH  
PHYSICAL TESTING

May 25, 1977

LABORATORY NUMBER: 577.329 A P.O. No. LO-8796

SAMPLE: One (1) Sample for  
Chemical Analysis

MARK: P-1  
DCPP #1 Steam Generator  
Nozzle Failure Analysis

DATE SUBMITTED: May 24, 1977

REPORT TO: Pacific Gas & Electric Company  
3400 Crow Canyon Road  
San Ramon, California 94583

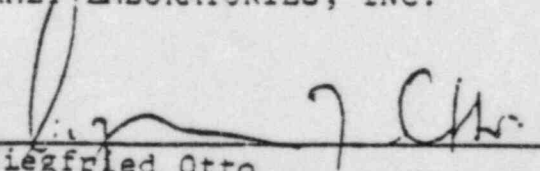
Attn: Mr. Steve Ivy

CHEMICAL ANALYSIS

			<u>Requirements</u>	
			A-106 Gr. B	
			<u>Min.</u>	<u>Max.</u>
Aluminum	(Al)	0.03%	Information	
Carbon	(C)	0.26%	-	0.30%
Chromium	(Cr)	0.14%	Information	
Copper	(Cu)	0.10%	Information	
Manganese	(Mn)	0.90%	0.29%	1.06%
Molybdenum	(Mo)	0.04%	Information	
Nickel	(Ni)	0.07%	Information	
Phosphorus	(P)	0.015%	-	0.04%
Silicon	(Si)	0.25%	0.10%	-
Sulfur	(S)	0.033%	-	0.05%
Titanium	(Ti)	0.003%	Information	
Vanadium	(V)	0.003%	Information	

Respectfully submitted,  
ANAMET LABORATORIES, INC.

By

  
Siegfried Otto  
Manager, Testing

3c  
jm

29  
LABORATORY CERTIFICATE

Report 411-77.55

Anamet Laboratories, Inc.

2827-79 STREET

BERKELEY, CALIFORNIA 94710

841-9771

May 25, 1977

ANALYTICAL  
CHEMICAL  
METALLURGICAL

HIGH TEMPERATURE  
APPLIED RESEARCH  
PHYSICAL TESTING

LABORATORY NUMBER:

577.329 B

P.O. No. LO-879806

SAMPLE:

Two (2) Samples for  
Chemical Analysis

MARK:

Weld - Side and Crown  
DCPP #1 Steam Generator  
Nozzle Failure Analysis

DATE SUBMITTED:

May 24, 1977

REPORT TO:

Pacific Gas & Electric Company  
3400 Crow Canyon Road  
San Ramon, California 94583

Attn:

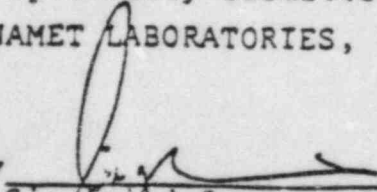
Mr. Steve Ivy

CHEMICAL ANALYSIS

		<u>Side</u>	<u>Crown</u>
Aluminum	(Al)	0.005%	0.004%
Carbon	(C)	0.08%	0.06%
Chromium	(Cr)	0.09%	0.06%
Copper	(Cu)	0.07%	0.08%
Manganese	(Mn)	1.17%	1.06%
Molybdenum	(Mo)	0.15%	0.12%
Nickel	(Ni)	1.00%	1.03%
Phosphorus	(P)	0.013%	0.012%
Silicon	(Si)	0.55%	0.60%
Sulfur	(S)	0.022%	0.022%
Titanium	(Ti)	0.01%	0.01%
Vanadium	(V)	0.03%	0.02%

Respectfully submitted,  
ANAMET LABORATORIES, INC.

By

  
Siegfried Otto  
Manager, Testing

3c  
jm



PROCEDURE SPECIFICATION FOR: Carbon Steel  
Welding to Nickel Steel piping, insert weld-  
GTAW (root) and SMAW (weld out).

BASE METAL: The base metal shall conform  
to the specifications for ASME, Section IX,  
CB to P-1 materials.

FILLER METAL: The filler metal shall conform  
to ASME Filler Metal Specifications Num-  
bers FFA-5.5 and FFA-5.18 for ferrous filler  
metal in Group Number F-4 and F-6.

Chemical composition of the weld deposit  
shall fall within the limits of Weld Metal  
Analysis Number A-12b.

SHIELD FOR TORCH SHIELD: Nominal composition  
Argon, 99.995% minimum purity (for GTAW  
process).

SHIELD FOR BACK-UP PURGE: Argon per Page 5  
(GTAW process).

ROOT WELDS FOR SET-UP: The GTAW process  
with filler metal type listed on Page 2 may  
be used with or without back-up purge in 1/16",  
or 1/8 inch diameter.

POSITION: The welding may be done in all  
positions.

HEAT AND INTERPASS: 200° F - 300° F pre-  
heat for SMAW weld out only and 200° F minimum  
interpass.

POST HEAT TREATMENT: 1100° F - 1200° F, 1  
hour per inch minimum (see job specifications  
cycle and thicknesses requiring post heat  
treatment).

BACKING STRIP: None.

TRAVEL SPEED: GTAW  $\frac{1}{8}$ " - 6" per min.  
SMAW 2" - 8" per min.

WELDING PROCESS: The welding shall be done by  
the GTAW insert root and SMAW weldout processes  
using manual equipment. The GTAW process may  
be used with the filler metal type listed on  
Page 2 for intermittent voids, look-in holes  
(peep holes) in the ring, or mismatch in the  
root set-up. If necessary, one complete pass  
may be made while holding the purge. GTAW  
welding shall be done using a non-consumable  
electrode of 2% thoriated tungsten, EMH-2.

BASE MATERIAL THICKNESS: This procedure  
is qualified to allow welding of material  
thickness between 3/16" and 1.5" (heat  
treated).

PREPARATION OF BASE MATERIAL: The edges  
or surfaces of the parts to be joined by  
welding shall be prepared by flame cutting,  
plasma arc, grinding, machining, or any  
combination of methods to essentially form  
the geometry of the weld shown on Page 2  
as detailed on the attached sketches and  
shall be clean of all oil or grease and  
excessive amounts of scale or rust.

ELECTRICAL CHARACTERISTICS: The current  
used shall be DC: GTAW Straight Polarity  
SMAW Reverse Polarity

JOINT WELDING PROCEDURE: The welding tech-  
nique, such as electrode sizes, and voltages  
and currents for each electrode, size of the  
welding tip and filler rods, shall be sub-  
stantially as shown on Page 2.

APPEARANCE OF WELDING LAYERS: The welding  
current and manner of depositing the weld  
metal shall be such that there shall be  
practically no undercutting on the side  
walls of the welding groove or the adjoining  
base material. See job specifications for  
specific undercutting limitations.

CLEANING: All slag or flux remaining on  
any bead of welding shall be removed before  
laying down the next successive bead of  
welding.

DEFECTS: Any cracks or blow holes that  
appear on the surface of any bead of welding  
shall be removed by chipping, grinding, or  
gouging before depositing the next success-  
ive bead of welding.



THE H. W. KELLOGG COMPANY  
A DIVISION OF FULLMAN INCORPORATED  
PIPING FABRICATIONRECOMMENDED FORM Q-1 MANUFACTURER'S RECORD OF WELDING PROCEDURE  
QUALIFICATION TESTS

★ Heat Treated

Specification No. P12b-P1-K1-F4-SHAW-6G Date 12/28/73  
 Welding Process GTAW & SHW Manual or Machine Manual  
 Material Specification A-508-CL2 to A-106-B of P-No. 120 to P-No. 1  
 Thickness (if pipe, diameter and wall thickness) 10" O.D. x 3/4" W  
 Thickness Range this test qualifies 3/16" thru 1 1/2" wall thk. 3" IPS and over  
 Filler Metal Group No. F-4 & 6 FLUX OR ATMOSPHERE  
 Weld Metal Analysis No. A-12b Flux Trade Name or Composition None  
 Describe Filler Metal if not included in Table Q-11.2 Inert Gas Composition Argon  
 or QN-11. Root-E70S-2, weld-out-E8018 Trade Name --- Flow Rate 20 CFH  
 For oxyacetylene welding—State if Filler Metal is sil- Is Backing Strip used? No  
 icon or aluminum killed. Preheat Temperature Range 200° F - 300° F  
 WELDING PROCEDURE Interpass Temperature Range 200° F minimum  
 Single or Multiple Pass Multiple Postheat Treatment 400° F/hr from 600° to  
 Single or Multiple Arc Single 1100° F, 1 hr at 1150° F, furnace cooled at  
 Position of Groove 6G (See Para. & Figs. Q-2 & Q-3, or QN-2 & QN-3) 400° F/H to 600°  
 (Flat, horizontal, vertical, or overhead; if vertical, state whether upward or downward) A/C

## FOR INFORMATION ONLY

Filler Wire—Diameter 1/8 x 5/32, 3/32, 1/8, 5/32 WELDING TECHNIQUES  
 Trade Name --- Joint Dimensions Accord with See Sheet 2  
 Type of Backing Argon amps --- volts --- inches per min. ---  
 Forehand or Backhand --- Current D.C. Polarity GTAW Straight  
 REDUCED SECTION TENSILE TEST (Figs. Q-6 and QN-6) SHAW Reverse

Specimen No.	Dimensions		Area	Ultimate Total Load, lb.	Ultimate Unit Stress, psi	Character of Failure and Location
	O.D.					
1	.498		.1948	14,000	71,900	Broke in Base Metal
2	.495		.1924	13,800	71,700	Broke in Base Metal

## GUIDED BEND TESTS (Figs. Q-7.1, Q-7.2, QN-7.1, QN-7.2, QN-7.3)

Type and Figure No.	Result	Type and Figure No.	Result
1	Satisfactory	3	Satisfactory
2	Satisfactory	4	Satisfactory

Results of Fillet Weld Tests, Fig. Q-9(c) N/AWelder's Name S. Selby Clock No. 35 Stamp No. 1

Who by virtue of these tests meets welder performance requirements.

Test Conducted by Magnaflux Corp. Laboratory—Test No. M/K D-1 ★  
per H. W. Kellogg

We certify that the statements in this record are correct and that the test welds were prepared, welded and tested in accordance with the requirements of Section IX of the ASME Code.

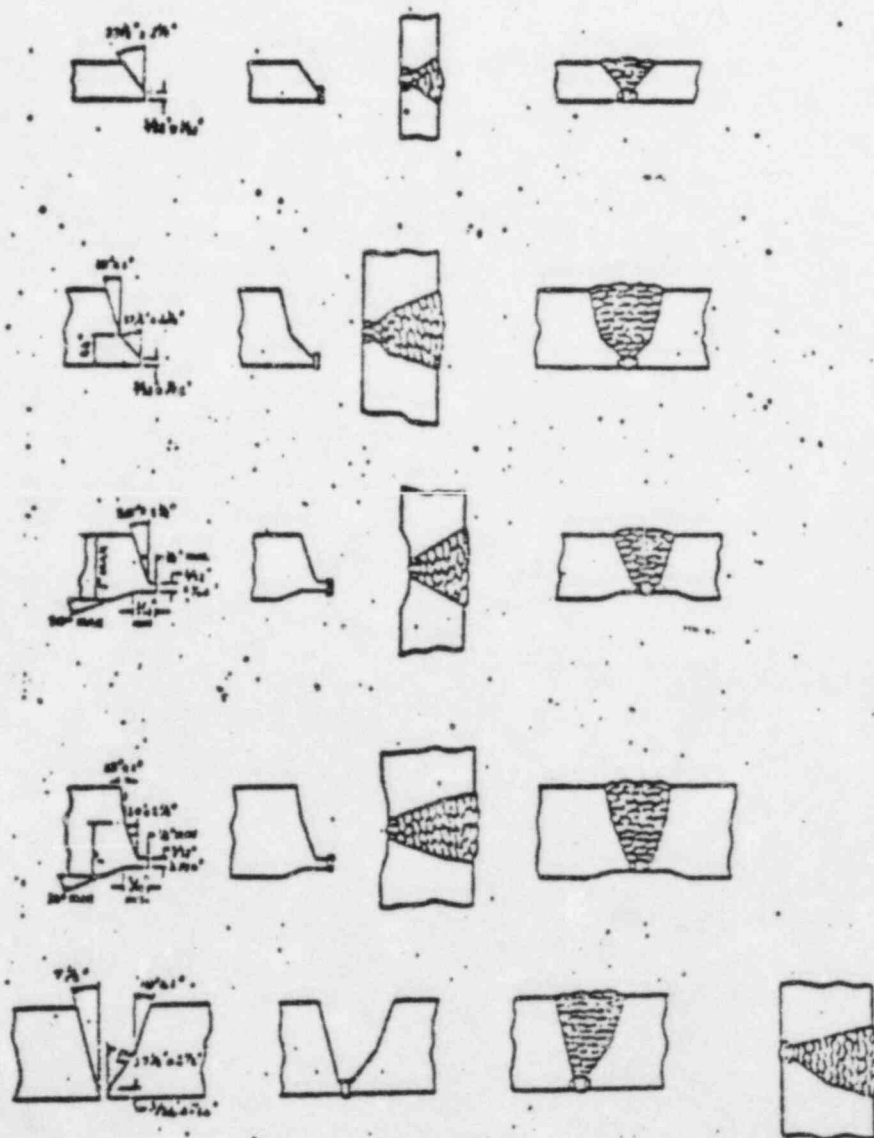
Signed The H. W. Kellogg Company  
(Manufacturer)Date 12/28/73By A. G. Fink

## Charpy "V" Notch Test Results:

H.A.Z. P12b = 66.5, 71.0, 71.0 = 69.5

Weld = 91.0, 85.0, 99.5 = 91.8

H.A.Z. P1 = 20.0, 27.0, 32.0 = 28.0



S. NO. AND PROCESS	FILLER METAL TYPE OPTIONAL	FILLER METAL SIZE OPTIONAL	AMPS	MAX. VOLTS	POLARITY	TORCH SHIELD & FLO RATE (MIN.)	TUNGSTEN SIZE AND POLARITY
GT TAW	E70S-2, or -6	INSERT 1/8 X 5/32	55-175	21	straight	Argon 20 CFH	1/8 or 3/32 diameter straight
CK TAW	E70S-2, or -6	1/16 3/32 1/8	60-120 70-150 100-140	14 17 20	straight	Argon 20 CFH	1/8 or 3/32 diameter straight
PLANCE TAW	E8018	3/32 1/8 5/32 3/16	65-110 100-165 140-220 180-275	27 31 34 36	reverse	—	—

\*NOTE: 1/16", 3/32", or 1/8" E70S-2 or E70S-6 filler metal may be used as necessary for tacking, filling intermittent voids, look-in holes (peep holes), or mismatch in the root set-up. If necessary, one complete pass may be made while holding the purge.

N/A : NOT APPLICABLE

NO. & SYSTEM		ISOMETRIC DRAWING NO.		DETAIL DRAWING NO.		SHEET NO.		
P. C. & E.		500146		N/A		100		
REPORTED BY	JOB NO.	DATE	CODE	ASME SECTION	CLASS	MARK NO.		
D.R.	7177	1/9/74		SECTION 1	SECTION 1	N/A		
PER. NO.	OPERATION	HOLD FOR AUTH. INSP.	HOLD FOR MFR INSP.	PROC. NO.	OPER.	DATE COMP.	MFR INSP. & DATE	AUTH. INSP. & DATE
	FIELD WELD #1 T9.7 SEE DR. 2450 ALSO 11-11-74							
1	CLEANING		✓	ESD 220	926	12/23/74	12/23/74	
2	FIT-UP	✓	✓	ESD 215	EX 126	12/23/74	12/23/74	
3	WELD ROOT PASS			CODE 207	6X	2/1/74		
4	PREHEAT SNAW ONLY (200° - 300°) USE CHART RELEVER		✓	CODE 207	117	3/2/74		
5	WELD COMPLETE			CODE 207	GX	3/2/74		
6	GRIND FOR R.T.			ESD 207	117	3/2/74		
7	VISUAL INSPECTION	✓	✓	ESD 215	117	3/2/74		
8	R.T. FINISHED WELD	✓	✓	ESD 207	K	1/21/75		
	-STRESS RELIEVE- SEE CHART NO-341	✓	✓	ESD 218	117	2/4/75		
	R.T. FINISHED WELD		✓	ESD 200	117	2-11-5		
	REMOVE DAMS		✓	ESD 214				
	PT FINISHED WELD		✓	ESD 211	N			
	FIELD WELD #2 12.2 RING TRUCKING 5/21/74							
	CLEANING		✓	ESD 220	117	5/21/74		
	FIT-UP	✓	✓	ESD 215	LP	5/21/74		
	WELD ROOT PASS	✓		CODE 200	LB	5/21/74		
	PREHEAT SNAW ONLY (200° - 300°) USE CHART RELEVER		✓	CODE 200	117	5/21/74		
	WELD COMPLETE			CODE 200	LB	5/21/74		
	GRIND FOR R.T.			ESD 207	415	5/21/74		
	VISUAL INSPECTION	✓	✓	ESD 215	HE	5/21/74		
	R.T. FINISHED WELD	✓	✓	ESD 207	ES	5/21/74		
	STRESS RELIEVE	✓	✓	ESD 218	117	5/21/74		
	R.T. FINISHED WELD		✓	ESD 214	GC	5/21/74		
	REMOVE DAMS		✓	ESD 214	GC	5/21/74		



Storck Friedrich  
 com: MRT/RO Eicher

PACIFIC GAS AND ELECTRIC COMPANY  
 STATION CONSTRUCTION DEPARTMENT  
 DIABLO CANYON PROJECT

Pg. 1 of 2

HISTORY OF STEAM GENERATOR 1-2  
 DIABLO CANYON PROJECT

March 28, 1977

The following information is provided per your request to indicate tests and transient conditions which influenced temperature and pressure on the secondary side of steam generator 1-2:

HYDRO TESTS

2/19/74 1st hydro - 1 cycle to 1356 psig @ 106° F (main steam and feedwater  
 nozzles capped at generator)  
 8/29/75 2nd hydro - 1 cycle to 1320 psig @ 88° F (RY's lifted and pressure had  
 to be reduced)  
 8/29/75 With Present Piping  
 3rd hydro - 1 cycle to 1356 psig @ 88° F

HOT FUNCTIONAL TEST

12/13/75	RCS heatup	100° F to 150° F
12/15/75	RCS heatup	150° F to 240° F
12/23/75	RCS heatup	261° F to 340° F, 62 psig to 103 psig
12/30/75	RCS heatup	350° F to 450° F, 119 psig to 407 psig
12/31/75	RCS heatup	450° F to 577° F, 407 psig to 750 psig
1/1/76	RCS heatup	577° F to 547° F, 750 psig to 1010 psig
1/4/76	pressure dip	1000 psig - 940 - 1000 psig, $\Delta T < 10^\circ F$
1/4/76	pressure dip	1000 psig - 925 - 995 psig, $\Delta T < 10^\circ F$
1/4/76	temperature drop	545° F to 523° F, 980 psig to 810 psig
1/6/76	temperature dip	524° F - 497 - 524° F, 824 psig - 651 - 824 psig
1/7/76	temperature dip	531° F - 505 - 512° F, 900 psig - 695 - 750 psig
1/8/76	temperature increase	508° F to 545° F, 714 psig to 983 psig
1/9/76	temperature dip	545° F - 503 - 547° F, 989 psig - 677 - 1000 psig
1/10/76	temperature dip	548° F - 526 - 542° F, 1000 psig - 829 - 973 psig
1/12/76	RCS cooldown	536° F to 115° F, 908 psig to 0 psig
1/15/76	RCS heatup	139° F to 327° F, 0 to 84 psig
1/17/76	RCS heatup	327° F to 528° F, 84 psig to 860 psig
1/21/76	temperature dip	525° F - 472 - 546° F, 840 psig - 501 - 982 psig
1/23/76	turbine roll	546° F - 476 - 501° F, 1000 psig - 525 - 663 psig
1/24/76	temperature increase	501° F to 545° F, 680 psig - 985 psig
1/28/76	turbine roll	547° F to 482° F, 979 psig to 563 psig
1/28/76	RCS cooldown	482° F to 255° F, 563 psig to 20 psig
1/29/76	RCS heatup	255° F to 546° F, 20 psig to 989 psig
2/4/76	turbine roll	546° F - 477 - 546° F, 986 psig - 537 - 993 psig
2/5/76	turbine roll	546° F - 485 - 532° F, 987 psig - 595 - 857 psig

# History of Steam Generator 1-2 Diablo Canyon Project

-2-

To: Spence Friedrich  
From: MRT/RD Etzler  
Pg. 2 of 2

## HOT FUNCTIONAL TEST-continued

2/7/76	tested safeties	956 psig - 864 psig several times $\Delta T < 10^\circ F$
2/10/76	tested safeties (S.G.'s 1-3 and 1-4)	552° F - 531° F four times 1042 psig - 868 psig
2/10/76	RCS cooldown	549.5° F to 100° F, 1008 psig to 0 psig

NOTE: During the days not listed above, the temperature and pressure were relatively stable. A detailed picture of temperature and pressure is available at the site and a drawing of the entire hot functional program. Steam pressure shown on the drawing is downstream of the main steam isolation valves and does not always agree with steam pressure in the steam generator.

## MINI-HOT FUNCTIONAL TEST

3/14/77	RCS heatup	115° F to 165° F
3/15/77	RCS heatup	165° F to 300° F, 0 psig to 52 psig
3/16/77	RCS heatup	300° F to 330° F, 52 psig to 88 psig
3/17/77	RCS cooldown	330° F to 100° F, 88 psig to 0 psig

## FEEDWATER TO STEAM GENERATOR

During both Hot Functional and Mini-Hot Functional, the auxiliary feedwater system was used for makeup to the steam generator. Feedwater temperature varied from 90° F to 110° F and flow rate varied from 0 to 300 GPM. We understand you have obtained water chemistry results from the plant chemist.

## THERMAL EXPANSION

During Hot Functional, thermal movement and potential component and piping interferences were closely monitored for each steam generator. Movement of steam generator 1-2 was in agreement with the movement of the remaining generators and acceptable by Engineering Department. However, insulation on the feedwater inlet line for steam generator 1-2 did come in contact with the missile barrier penetration, whereas feedwater lines on the other steam generators did not. The insulation cover was slightly indented, but not enough to induce a restriction to the movement of the feedwater line with the steam generator. Results of the thermal expansion test during Hot Functional are on file at the plant and General Office.

R. H. WOOD  
Startup Department

Edited: VJR  
cc File



## ANALYSIS OF ELECTRON DIFFRACTION PATTERNS

Gold Calibration for Camera Constant

<u>Ring Dia. cm</u>	<u>dÅ</u>	<u>K</u>
2.30	2.355	5.42
2.65	2.039	5.40
3.75	1.442	5.41
4.40	1.230	5.41

$$K = 5.41$$

Pattern From Extracted Oxide

<u>Ring Dia. cm</u>	<u>dÅ</u>	<u>Intensity*</u>
1.3	4.15	S
1.5	3.60	W
2.0	2.70	S
2.15	2.52	M
2.40	2.25	M
2.70	2.00	W
3.10	1.74	M
3.30	1.63	W
3.50	1.55	VS
4.00	1.35	S
4.40	1.22	W
4.70	1.15	M

\*VS - Very Strong  
 S - Strong  
 M - Medium  
 W - Weak

\*  $\alpha\text{Fe}_2\text{O}_3$  Hematite



EXHIBIT 7 TO  
ATTACHMENT 1



**Pullman Swindell**  
Division of Pullman Incorporated

441 Smithfield Street  
Pittsburgh, Pennsylvania 15222  
(412) 562-7000  
Telex 866 500

Metallurgical Examination  
Cracked Field Weld No. 212  
Feed Water Pipe to Nozzle; Steam Generator 1-2.


Pacific Gas & Electric Co.  
Diablo Canyon Nuclear Project

by

Pullman Power Products Co.  
Williamsport, Pennsylvania

Date: 6/23/77

Prepared by:

  
R.H. Caughey  
Chief Metallurgist

## INTRODUCTION

On March 17, 1977, during hot functional testing, a water leak was discovered in the butt weld (Field Weld No. 212, line K16-555-16IV) which joined the 16 inch diameter feed water pipe to the No. 4 nozzle on steam generator 1-2. The line pressure and temperature at this time was approximately 90 p.s.i. and 300°F respectively.

Subsequent on site non-destructive testing by both ultrasonic and radiographic means (by both Kellogg and P.G. & E. representatives) led to the conclusion to cut out and totally replace this weld. A satisfactory, acceptable replacement was completed during the period March 19-25, incl. 1977.

A summary of these events are described in Kellogg's report dated April 12, 1977 prepared by J.P. Runyan, Field QA/QC Manager. A copy of this report is attached (Appendix I, pages 11-15)

Incident with this replacement a spool piece was salvaged for subsequent laboratory examination by P.G. & E. It contained a conveniently large portion of the weld deposit and a representative portion of both the nozzle and pipe components. A sketch which describes the dimension of this piece; also the crack path and the extent of the crack relative to the weld deposit is included with the Kellogg report referenced above (Appendix I).

Kellogg, for the purpose of a laboratory examination, requested P.G. & E. to furnish representative specimens cut from this spool piece. Two were received.

The report which follows covers a description of these samples and the results and conclusions determined from this investigation.





## PROCEDURE & RESULTS OBTAINED

### A. History

Pertinent to this investigation was a complete review of the records appertaining to field weld joint No. 212, namely; the weld procedure; welder qualification records, heat treatment; material certifications; nondestructive examination personnel qualification and N.D.E. results. The following information was gleaned from the recorded documents on file with respect to this weld:

#### 1. WELD HISTORY REVIEW

Field weld 212 was fit-up and tack welded on Friday May 18, 1974. On Monday May 21, 1974 the tack welds and insert were removed. A new insert was installed on May 22, 1974. It is not documented as to why the original insert was removed, however, discussions with field inspectors who were on site at the time indicate that surface rust may have occurred over the weekend and the insert was removed to reclean the weld prep and install a new insert prior to consuming the ring.

The root pass was made on May 22, and accepted visually May 23, 1974. Welding proceeded following Q.C. acceptance and was completed on May 24, 1974.

The weld surface was ground and final visual inspection completed on May 28, 1974.

Other than replacing the insert no unusual or out of the ordinary circumstances were recorded.

#### 2. PREHEAT AND POSTHEAT

A review of the heat charts on F.W. 212 indicate that the weld area was preheated to 200°F min. prior to tack welding and subsequent welding.

The weld was preheated on May 22, 23, and 24. The heat was turned off at the end of each shift allowing the weld to cool.



2. PREHEAT AND POSTHEAT (Con't)

Following completion of the weld on May 24, the weld was allowed to cool and no further heating was performed until post weld heat treatment on June 24, 1974.

Preheat and post heat records from other welds of the same type indicate a similar history. NOTE: Other welds reviewed were the remaining three feedwater nozzle to pipe and all four main steam nozzle to pipe and all four main steam nozzle to pipe welds in Unit I and the main steam nozzle to pipe welds in Unit II. The feedwater nozzle to pipe welds are not welded in Unit II.

3. WELDING PROCEDURE AND WELDER PERFORMANCE QUALIFICATION RECORDS

Field Weld 212 was made using weld procedure number 200. The procedure was reviewed to assure compliance with ASME Section IX. No deviations were noted. The results of the procedure qualification tests were evaluated. Included were bend tests, tension tests, and Charpy V Notch tests. All results were acceptable.

Welder performance qualification records were reviewed. The records were in order and the welder was found to be properly qualified. In addition, records of other welds performed by the same welder were reviewed. It was determined that his performance record was good. There was no reason to suspect that welders capability or performance was below standard.

4. MATERIAL CERTIFICATIONS

Material Certifications of the weld material and pipe-side base material was reviewed for compliance with material specifications and job specifications, including supplementary requirements. All were found to be in compliance. The nozzle base material certifications were not available for review by M.W. Kellogg.



5. NONDESTRUCTIVE EXAMINATION PERSONNEL, PROCEDURES AND REPORTS

The original radiographs of F.W. 212 were reviewed. There was an area along with nozzle side of the root with greater density than other areas of the weld. This area is typical of other similar welds. The higher density is caused by the nozzle counterbore which is approximately 1/4" wide. There was no evidence of linear indications in the area where the crack occurred. Review of the radiographs of other similar welds did not show evidence of linear or crack-like indications.

B. Metallurgical Examination

A description of the two (2) specimens received from P.G. & E. follows:

One represented by about 1/2 inch of the circumference contained the full depth, "front" face of the crack as viewed toward the pipe component. An enlarged photograph showing the "front" face is displayed in Fig. 1 Page 8. This surface had been fully protected from post atmosphere affects by the application of a lacquer. The coating was removed for photographing.

The second (also about 1/2 inch of circumference) contained the full cross section of the weld joint. It had apparently been removed from a location where cracking had not occurred. A photograph of this sample is not included. It will be discussed with relation to microscopic examinations which were conducted on both specimens.

Photomicrographs of both specimens are displayed in Fig. 2 and 3\*respectively. Both represent the existing metallic structure in the locus of the weld root on the nozzle side of the weldment.

Noteable in Fig. 1, or the "front" face of the crack cross section is the distinct, variously "shaded" areas. There appears to be four in number as illustrated in the accompanying sketch.

\* Figure 2, Page 9  
Figure 3, Page 10

## DISCUSSION

Fractured or cracked areas showing these distinctions are typically associated with fatigue failures caused by the imposition of cyclic stress.

From this photograph the clear indication is that crack initiation took place in the root of the weld in the forged nozzle and ultimately propagated to failure through the weld deposit in at least 3 stages as shown in the sketch. (The cycling involved could have resulted either from multiple hydro-testing or from cyclic exposure during hot functional testing. These facts were not contained in the reported history cited above).

In such cases the causal phenomenon is usually a pre-existing discontinuity which may be either an inherent metallurgical plane of weakness or a physical crack like defect.

In the photograph of the fractured surfaces (Fig. 1) the first distinct area is the root of the weld joint. Close visual examination by macroscopic means showed sporadic small areas (light areas in the photograph) which were relatively bright or "shined".

Fig. 2 is a photomicrograph of the metallic structure which was found to exist in one of these areas. The indication is that these are associated with remanent, weld bevel "land" material of the forged nozzle component. The suggestion is that, crack initiation stemmed from this locus; also that the initiating defects were localized, incipient root cracks through the land area.

What is not clear from this examination is the cause for the defects. In the history of the field fabrication of this particular weld (cited above) the fact that repetition preheating and cooling was permitted to take place on four separate occasions before the weld was completed, could be one factor. Thereby inordinately high thermal stresses could have resulted in incipient cracks.





## DISCUSSION (Con't)

Another contributing factor under these circumstances (which cannot be overlooked) could be the relative hardenability of the forged nozzle material the reported composition of which was a nickel, molybdenum bearing steel or A.S.T.M. A508-CL2. This is an air hardening composition. It is possible due to the noted in attention especially to cooling affects, following the root pass welding, that, air hardened, crack sensitive microstructures could have been effected and sporadic cracks on cooling promoted in the initial land area. (Microstructure affects in the specimens examined would, of course, be obscured by the subsequent stress-relief heat treatment.)

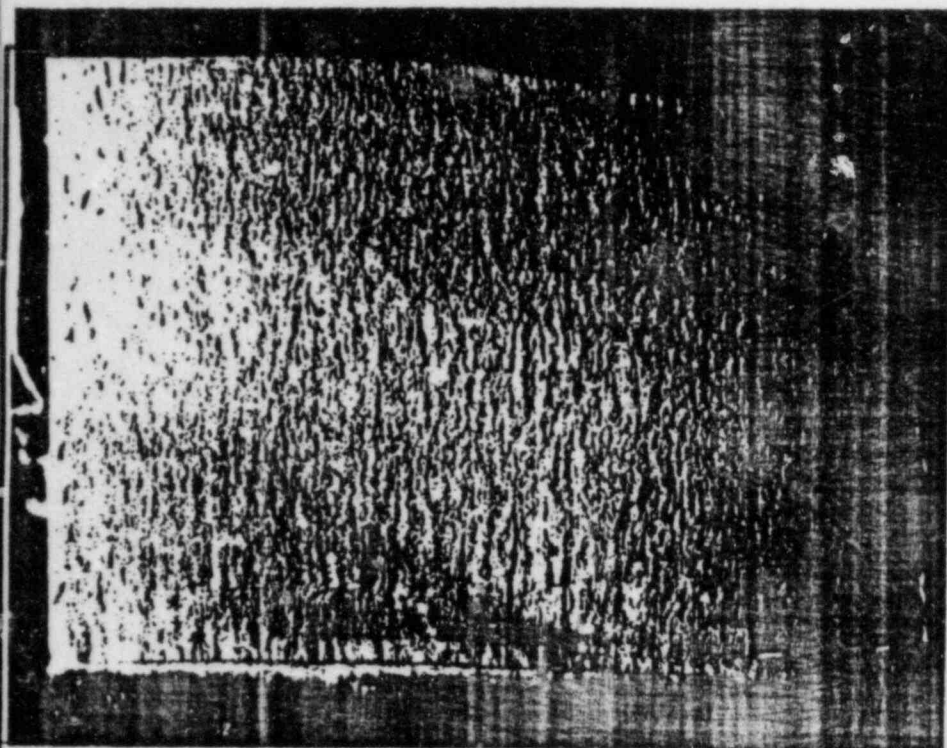
Understanding this weld (NO. 212) was one which was completed under restrained conditions (or as a closure weld) the possible circumstances i.e., in attention to preheating and cooling and metallurgical transformations, would augment the cracking tendencies.

Fig. 3 or the photomicrograph of the uncracked weld joint (localized at the root weld in nozzle forging) shows the normally expected microstructure; good penetration; and no evidence for lack of fusion or cracking. Certain evidence of small cracks are shown to exist in the I.D. surface of the forging. But there is no reason to believe these or like defects could have contributed to ultimate failure, once the weld was completed and stress relieved.



### CONCLUSIONS

1. The crack failure in the FW212 resulted from crack like defects which apparently existed in the weld bevel land.
2. The cause for the crack defects is not clear. They are believed, however, to have resulted from the welding process and probably portend the affects of repetitive preheats and cooling applied before welding was completed. Imposition of high thermal stress, especially in the early stages of welding, is a plausible explanation.
3. Ultimate failure through the weld deposit was due to a cyclic fatigue stress phenomenon, which it is suggested was the consequence of repetitive loading due either to preliminary hydrotesting or inherent in the hot functional testing program.
4. The fact that cracking was only discovered during the hot functional testing, and not evident from preliminary in-process inspections i.e., ultrasonic and radiography (probably due to the inherent insensitivity of these procedures to indicate small incipient cracking) the recommendation is hereby made that all like welds should be reinspected upon the completion of functional testing and before service in order to discover any evidence of crack propagation or enlargement; not withstanding the fact that in process documents failed to disclose the possibility of similar incidents.

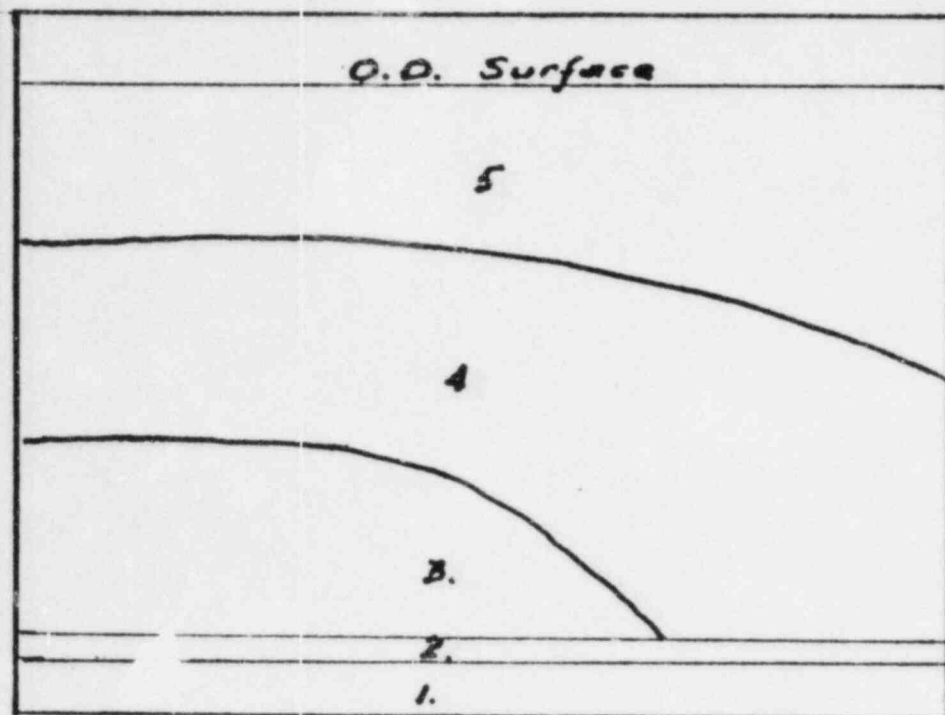


As Received

5X

Photograph

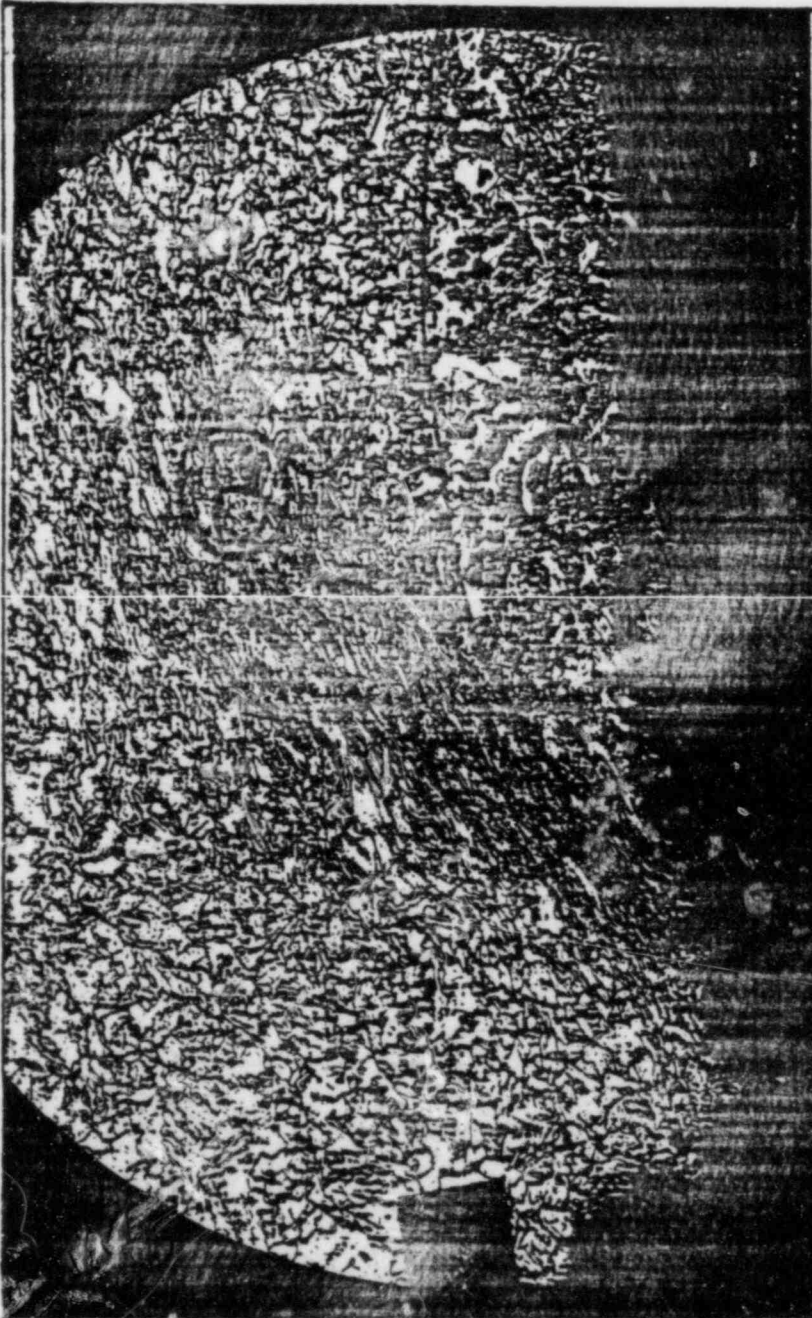
Crack "Front" Face; Viewed Toward  
Pipe Component.



LEGEND

1. K-insert, weld reinforcement.
2. Remanent weld bevel land; probable crack initiation region. Refer to photomicrograph, Fig. 2.
3. 2nd-crack stage.
4. 3rd-crack stage.
5. Final stage, thru wall penetration.

Fig. 1



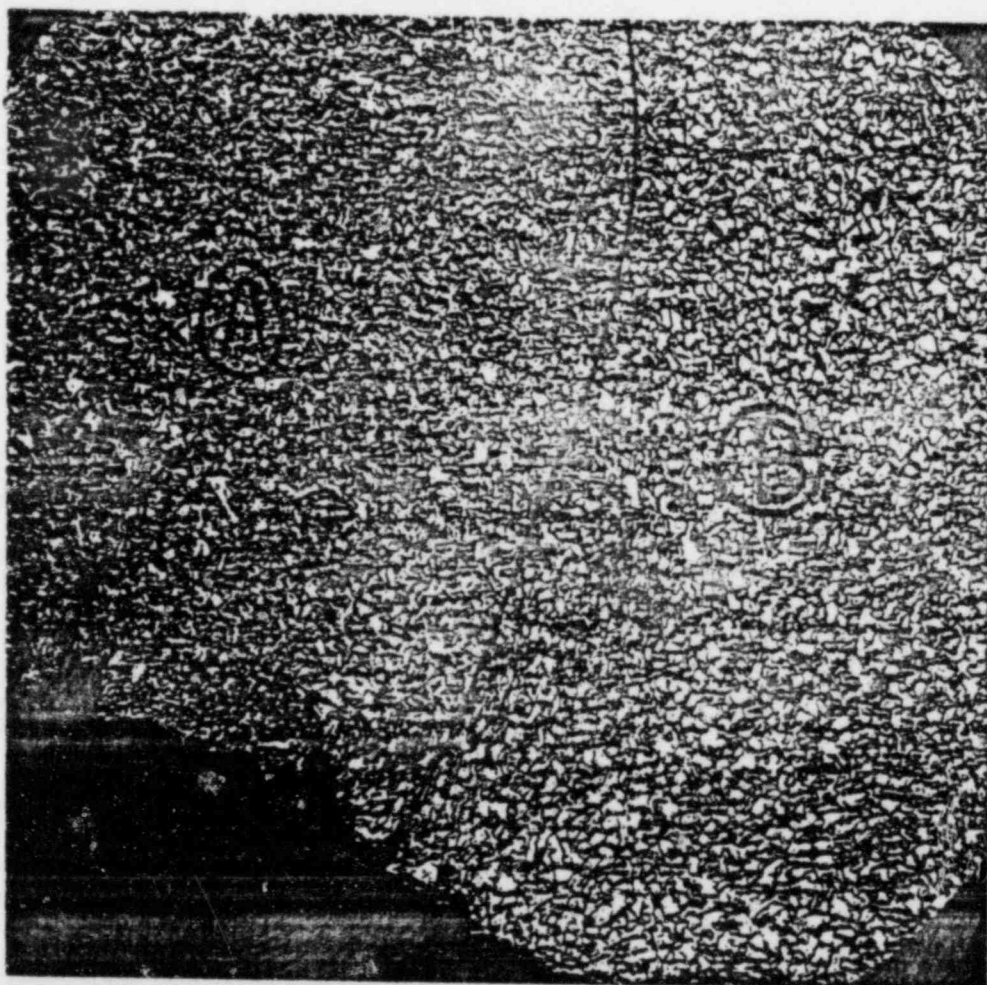
- A. Remanent weld bevel land in forged nozzle.
- B. K-insert, weld I.D. Reinforcement.

2% nital etch

250X

Photomicrograph  
Crack Evidence in Initial Weld Bevel Land





2% Nital Etch

100X

- A. Nozzle Forging
- B. K-insert root weld and  
I.D. Reinforcement

Photomicrograph; Root Weld, Nozzle  
Forging Component

Fig. 3



**Pullman Swindell**  
Division of Pullman Incorporated

## APPENDIX I

DIABLO CANYON NUCLEAR PROJECT

JOB # 7177

SPEC # 8711

QUALITY ASSURANCE REPORT

OF

Crack in Steam Generator Feed  
Nozzle to pipe weld  
(Generator No. 1-2)

THE M.W. KELLOGG CO.  
Avila Beach, Ca.

PREPARED BY

J. P. Runyan  
J. P. RUNYAN

Field Q.A./Q.C. Manager

DATE

April 12, 1977

QUALITY ASSURANCE REPORT  
OF  
CRACK IN STEAM GENERATOR FEEDWATER NOZZLE  
TO PIPE WELD (GENERATOR NO. 1-2)

PURPOSE

To document events from the time of discovery of the leak until the completion of the repair.

SCOPE

This report covers the on site findings, the review of documentation including preheat and postheat charts, radiographs from the defective weld, the subsequent procedures for removing the defective area and performing the repair.

FINDINGS

On Thursday, March 17, 1977 a leak was observed in field weld 212, line K16-555, by P.G. & E. We were advised of the leak on that date.

The weld in question is where the 16" feedwater pipe ties into nozzle #4 on Steam Generator 1-2.

Visual observation revealed a weep type leak which occasionally sprayed a fine stream of water. When observed with a 10X magnifying glass there appeared to be a small intermittent linear indication approximately 3/8" in length in the center of the weld running around the pipe. Magnetic particle examination of the area with a D.C. converted yoke did not show evidence of a linear defect.

P.G. & E. requested that we grind the area. As grinding proceeded, the indication opened to reveal a linear defect approximately 2" long when the weld crown was flush with the pipe surface. At this point the grinding was stopped. We were requested to perform an Ultrasonic examination to determine the extent of the indication. The pipe temperature was approximately 180° F which made it impossible to perform U.T. with the standard transducers on site.

We were then instructed to "hold" until P.G. & E. Engineering Research arrived with their U.T. equipment and high temperature transducers. The weld was radiographed at this time. The radiograph revealed evidence of a linear indication which appeared to be approximately 6" in length.

On Friday, March 18, 1977 P.G. & E. Engineering Research arrived and performed Ultrasonic examination of the weld. They reported that there was a crack which appeared to extend approximately 2/3 of the distance around the weld. Based on their findings it was determined that the weld would be cut out and replaced.



## WELD REMOVAL

The weld was cut by grinding approximately 1/2" from the center line on the nozzle side and at F.W. 503 on the pipe side. The end of the pipe was then cut to remove a ring which included most of the weld and approximately 4" of pipe.

The piece was examined visually and by liquid penetrant on the O.D. and I.D. and a sketch made to reflect the observations. (Sketch Attached)

The piece was shipped to P.G. & E. research lab on March 20 for analysis.

## REPAIR

A piece of 16" pipe was removed from stores to replace the piece which was cut out. The pipe end preps, gamma hole and vent were machined in the P.G. & E. machine shop on March 19, 1977. The nozzle end prep at F.W. 212 and pipe end at F.W. 503 were ground in place on March 20. A liquid Penetrant examination was performed at that time to assure complete removal of any indications.

The new piece was moved into place and fit up on March 20. Preheat was applied prior to tack-up, (Ref. Chart # 547). After the fit up was approved by M.W. Kellogg Q.C. both roots were welded. Magnetic particle inspection was performed. Following magnetic particle acceptance two additional passes were welded in each weld.

Radiography was performed on March 21, 1977. The radiographs of F.W. 503 pipe to pipe were acceptable. F.W. 212, pipe to nozzle, had excessive porosity at the window closures. These areas were ground, rewelded and re-radiographed. One area had excessive porosity, the other had a linear indication approximately 1/2" long and an area which appeared to be suck back.

At this time, March 21, 1977, P.G. & E. Q.A. placed a "hold" on all work until their Engineering could review the total program and process appropriate paperwork. "This hold was not because of the difficulty in welding the window closures".

No work was performed on March 22, 1977.

On March 23, 1977 an on site meeting was held to review findings, work to date, and procedures for completion of the repair. Attending the meeting were representatives from P.G. & E. Engineering, Q.A., General Construction, Division, Westinghouse, M.W. Kellogg site Manager, Corporate Q.A. Manager, and the writer.

- A. It was determined that the procedure as outlined by D.R. 3366 was acceptable except that the preheat would be raised to 300° Min. for completion of the welding.
- B. The problem of making an acceptable closure weld was discussed. It was determined that the positive pressure maintained in the system was too high and that water remaining in the generator caused vapor to be carried out the window openings.

C. P.G. & E. Division agreed to drain the generator and reduce the purge pressure until the closure welds could be made.

On March 24, 1977 the defective areas at the closures were ground out and Magnetic particle performed to assure complete removal. No welding was performed since P.G. & E. had not removed the Hold. Preheat was being maintained through the "hold" period.

On March 25, 1977 the hold was released. The windows were closed and radiograph made. Both areas were acceptable and welding continued. The weld was completed on the afternoon of the 25th. Radiographs were shot in the hot as welded condition. No rejectable indications were noted. Post weld heat treatment was then performed.

On March 26, 1977 following post weld heat treatment, the weld was ground, re-radiographed, liquid penetrant inspected and ultrasonically examined. No rejectable indications were noted. The vent line and gamma plug were completed, inspected and the system turned over to division for resumption of testing.

#### SUBSEQUENT INVESTIGATION

Because of the unknown origin of the crack it was determined that other welds of the same type and welded using the same procedure should be reviewed to determine if the defect could be a generic nature.

Each of the remaining three feedwater nozzle welds and the four Main Steam nozzle welds were ultrasonically examined. The original radiographs were reviewed and the feedwater nozzle welds were re-radiographed. There was no evidence of like indications in any of the welds and no evidence of change in any of the noted, acceptable, indications since the original inspections.

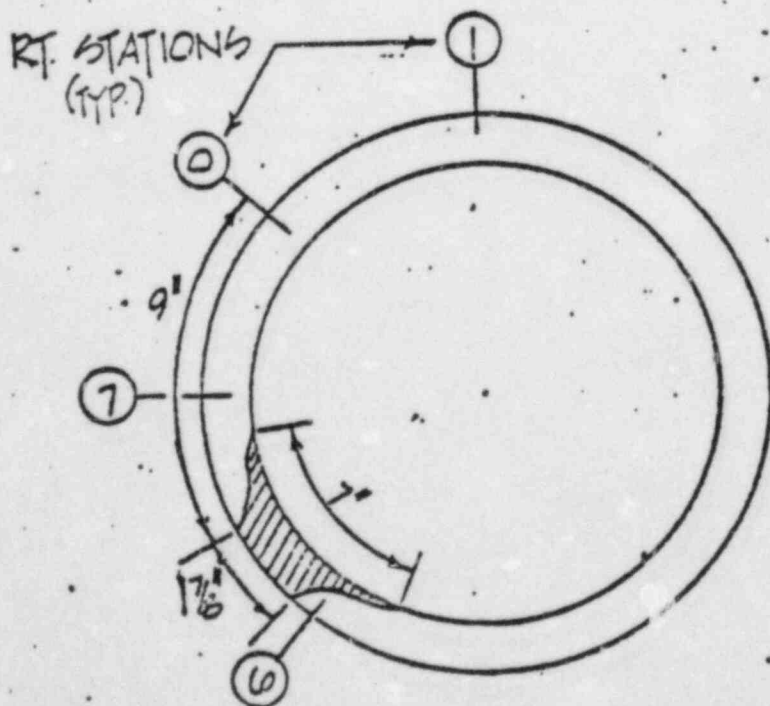
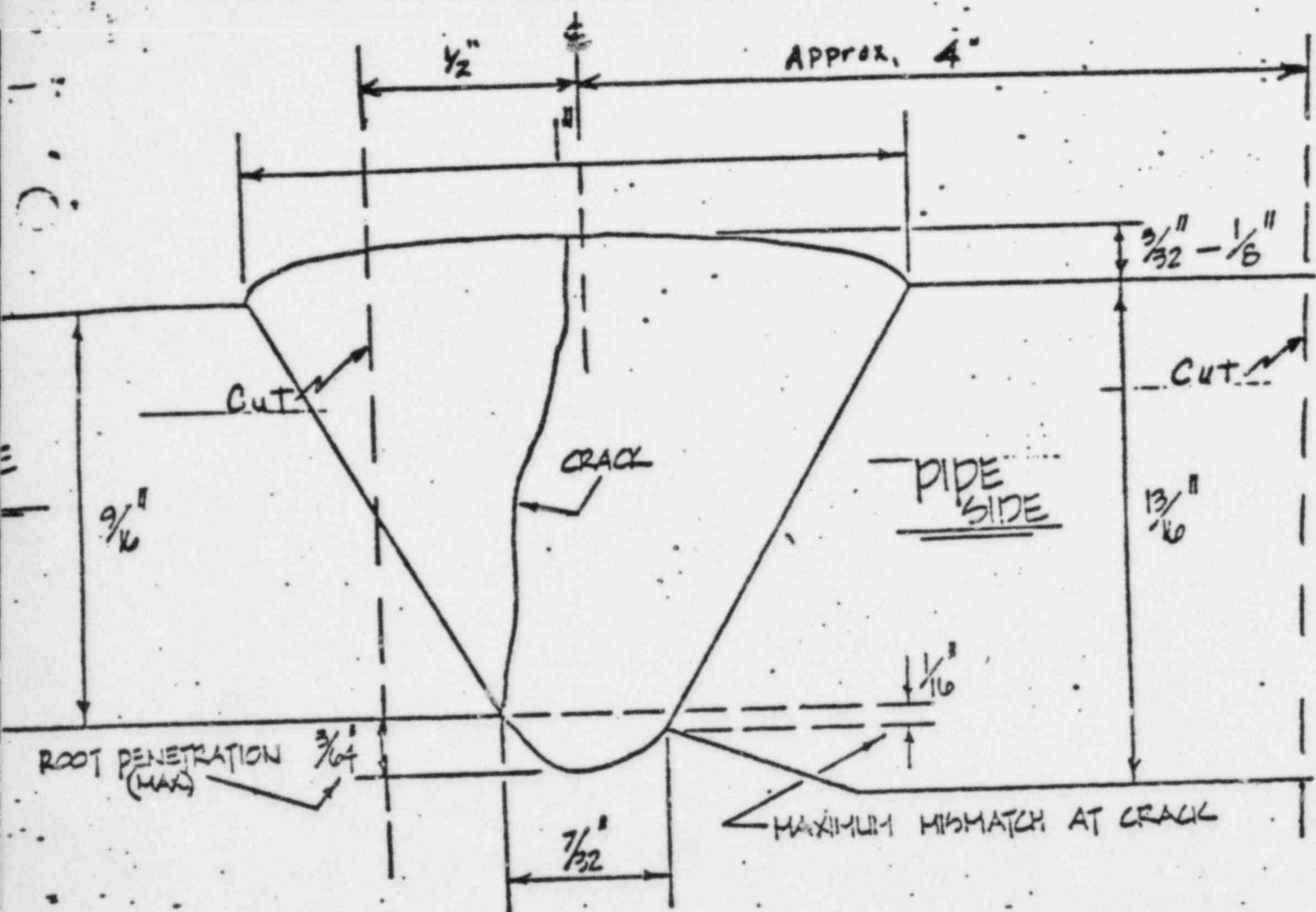
There was, however, an indication noted in F.W. 197 on generator 1-1. The indication was present on the original radiographs but had not been noted on the radiographs report. A second set of radiographs were on file for F.W. 197. This set also exhibited the same indication, however, the radiographic report did reflect the condition as being a drop thru. The indication had apparently been evaluated and determined acceptable at the time.

Because of the problem with F.W. 212 it was determined that any questionable situation should be resolved. It was therefore determined that the drop thru should be removed. A D.R. was initiated and the repair was made. The indication was removed by grinding. No welding was required. (See D.R. 3370 Attached)

#### SUMMARY

To date the metallurgical analysis has not been completed of the crack sample. We had requested, from P.G. & E., a piece of the defective material for our own analysis. This was received on Thursday, April 7, 1977 and forwarded to E. F. Gerwin in Williamsport.

It is my belief that the crack was peculiar to F.W. 212 only and not of a generic nature. Therefore, at this time we are assuming that no further repair will be required and that when the disposition of D.R. 3370 is completed the subject will be closed.



# NOTE:

1. WALL THICKNESS MEASR. ARE APPROXIMATE
2. REF. TO DR. 3366 REV. A
3. STEAM GENERATOR 1-2  
F.W. #212  
NO. 500146  
LINE # K16-555-16 IV

DR 3366 Rev I  
ATTACHMENT # 2

PULLMAN POWER PRODUCTS

EVALUATION OF DISCREPANCY REPORT IN RELATION  
TO REQUIREMENTS OF 10CFR PART 21

D.R. # 3453 Rev1, to which this page is attached and  
become a part of, (is not, may be ) considered to be reportable  
to the NRC under the requirements of 10CFR Part 21.

PULLMAN POWER PRODUCTS

by [Signature]

Pacific Gas & Electric Company ( does, does not ) consider this  
discrepancy reportable and ( has reported, will not report ) the  
above discrepancy.

This form returned to Pullman Power Products on 9/12/77  
DATE

PACIFIC GAS & ELECTRIC CO.

by R.D. Tyler



## PULLMAN POWER PRODUCTS

EVALUATION OF DISCREPANCY REPORT IN RELATION  
TO REQUIREMENTS OF 10CFR PART 21

D.R. # 3453, to which this page is attached and  
become a part of, (is not, may be ) considered to be reportable  
to the NRC under the requirements of 10CFR Part 21.

PULLMAN POWER PRODUCTS

by *J. P. [Signature]*

Pacific Gas & Electric Company ( does, (does not) consider this  
discrepancy reportable and ( has reported, (will not report) the  
above discrepancy.

This form returned to Pullman Power Products on 8/29/77.  
DATE

PACIFIC GAS &amp; ELECTRIC CO.

by *R. D. [Signature]*

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

Page 1 of 3

D.R. NO. 3453 Rev. 1 Rev. 2  
ISO. NO. 500146 Shr. 4 of 5  
UNIT NO. 1  
CODE NO. N/A

## DISCREPANCY REPORT

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16": Spool Piece 500; El. 154'-4 1/2"; Area G; Sect. 1

EXPLANATION OF DISCREPANCY: Reference P.G.&E. Drawing 500146, Rev. 9B

Per P.G.&E.'s request, an internal inspection of F.W. 244 on Steam Generator 1-4 Feedwater Nozzle will be performed. The line entry will be made in the following manner, see Recommended Disposition.

FOR INFORMATION ONLY

### RECOMMENDED DISPOSITION:

1. Measure 2" from F.W. 244 center upstream on pipe (Spool Piece 500) circumscribe the pipe using a wrap around, then prick punch the scribe line.
2. Using air-arc, cut the pipe all around maintaining less than a 40° total bevel to within approximately 1/8" from the I.D. (See Sketch #1).
3. Using a thin blade grinder, cut through the center of the bevel, then dress up both bevels (See Sketch #1).
4. Jack pipe away from cut on upstream side per Superintendent's instruction. Install a dam in nozzle beyond F.W. 244 to prevent air from entering Steam Generator.
5. Clean and inspect F.W. 244 internally as directed by P.G.&E. (Con't on Page 2)

Approved By: M.W.K. Field Q.A. Mgr. J.P. Runyan Date 8/26/77 Customer R.D. Etley Date 8/29/77 *MUR 3553*

FINAL DISPOSITION: ☒ In Accordance With Above

☒ Other (explanation and approval required)

Work Completed Insp: S.B. Burke Date: 11-2-77

Work Completed Insp: S.B. Burke Date: 11-2-77

### EXPLANATION (IF NECESSARY):

Revision 1 - Delete Step 6 in its entirety (access was achieved). Refer to Attachment #1 for results of initial inspection and recommended disposition.

Revision 2 - See Page 3 of 3 and Attachment #3.

Rev #2 J.P. Runyan 9/7/77 R.D. Etley 8-26-77  
Rev #1 J.P. Runyan 8/3/77 R.D. Etley 8/29/77 *MUR 3553*

STEPS TO PREVENT RECURRENCE ☒ Not Applicable MR

Field Q.A. Manager

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Cust. Mgr. ☐ Receiving ☐ Field Inspector

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

Page 2 of 3

D.R. NO. 3453 Rev. 1 RE  
ISO. NO. 500146 Sht. 4 of  
UNIT NO. 1  
CODE NO. N/A

## DISCREPANCY REPORT

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16"; Spool Piece 500; El. 154'-4½"; Area G; Sect. I

### RECOMMENDED DISPOSITION: (CON'T)

6. If access to F.W. 244 I.D. cannot be achieved, proceed with alternate instructions (See Sketch #2).
7. Upon completion of inspection, remove dam, grind bevel to facilitate fit-up as needed, clean and fit to ESD-215 and ESD-220.
8. Weld out using F.W. 549 and procedure 4/5.
9. Inspect and NDE to ESD-215 and ESD-206.
10. Add all information to process sheet and isometrics.

FOR INFORMATION ONLY

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

Page 3 of 3.

## DISCREPANCY REPORT

D.R. NO. 3453 REV 2  
ISO. NO. 500146 Shr. 4 of 5  
UNIT NO. 1  
CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16"; Spool Piece 500; El. 154'-4 1/2"; Area G; Sect. I

D.R. 3453, Revision 2

### EXPLANATION OF DISCREPANCY:

During the performance of grinding per D.R. 3453, Revision 1, Step #5, four (4) indications of heavy drop through with roll over were removed along with all of the existing root penetration. Re-inspection revealed a linear indication 11" long on the fusion line to the Feedwater Nozzle. Further grinding, to a maximum depth of .110", removed the linear indication (see Attachment #3).

## FOR INFORMATION ONLY

### COMMENDED DISPOSITION:

Add the following to Step #5 and delete original Steps #7, 8, 9 and 10. Perform the following repair:

- Grind the bevels to facilitate Fit-Up of F.W. 549.
- M.T. I.D. of F.W. 244 using ESD-247 coil method, and clean grooved root pass at F.W. 244 per ESD-215.
- Remove the dam from the nozzle at F.W. 244.
- Preheat F.W. 244 to a minimum of 250°F; interpass not to exceed 550°F maximum. Maintain this preheat until stress relief begins.
- Weld to procedure 200 (F.W. 244 R1), using E70S2 wire with the GTAW process, to conform to blended configuration of the balance of the weld. I.D. ONLY
- Grind and polish the added weld metal to blend with base metal surfaces.
- Inspect and NDE to ESD-215 and PG&E DER TO PERFORM L.P. AT PREHEAT-TEMP.  
FURNISH MWK WITH COPY OF P&E CERT.

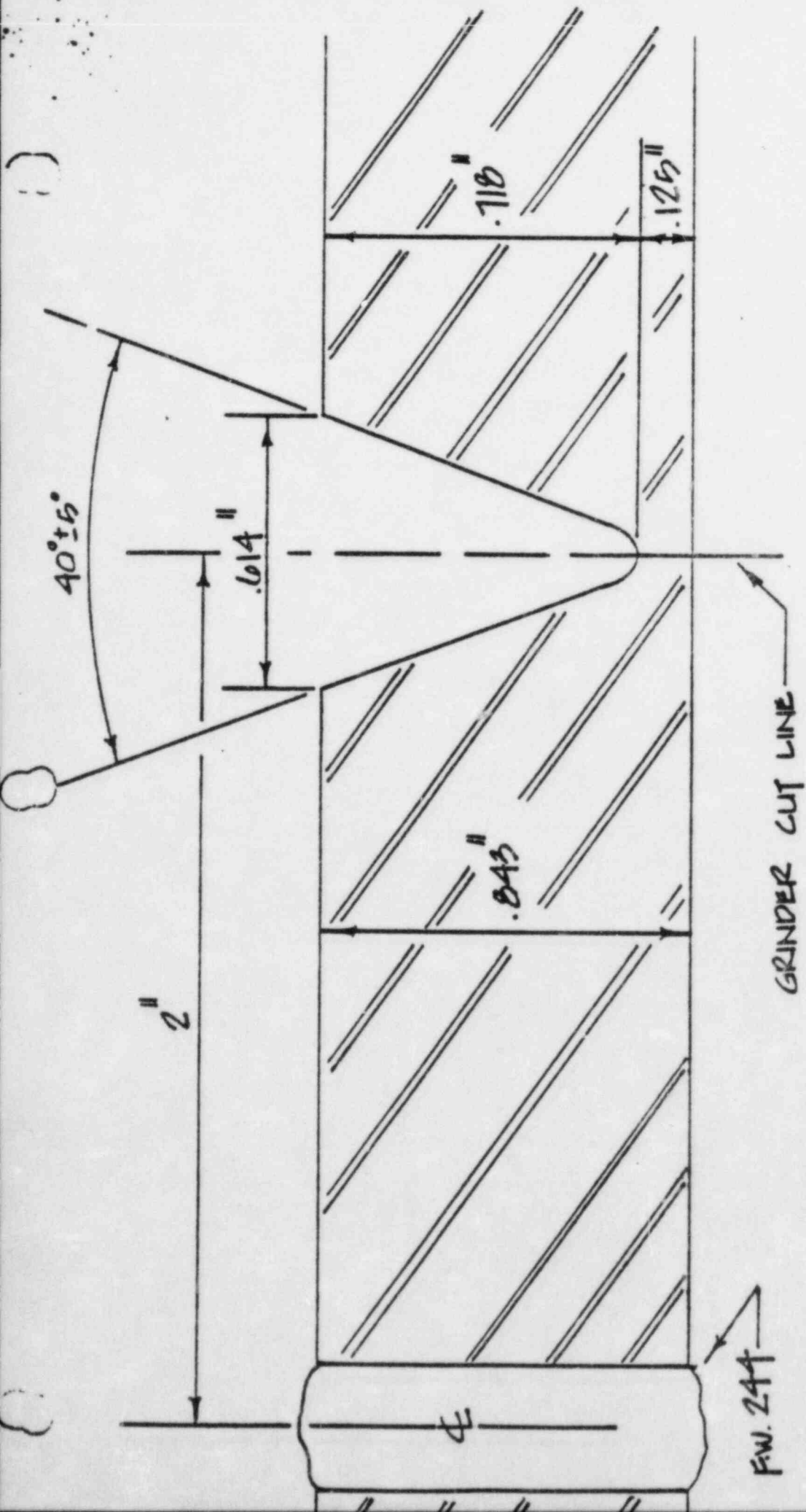
NOTE: Proceed with D.R. 3456, and X-ray weld, hot, for information only.

- Upon completion of inspection, remove dams from F.W. 549 pipe side, clean and fit F.W. 549 to ESD-215 and ESD-220.
- Weld out using F.W. 549 with procedure 4/5 A.O.
- Grind F.W. 549 for X-ray.
- Inspect and NDE F.W. 549 to ESD-215 PG&E DER TO PERFORM L.P. AT PREHEAT-TEMP.  
FURNISH MWK WITH COPY OF P&E CERT. 9/30/77 11 AM

NOTE: X-ray weld hot for information only.

- Stress relieve at F.W. 244 R1 and F.W. 549 1100°F to 1150°F per ESD-218.
- NDE F.W. 244 R1 and F.W. 549 to ESD-207.
- " " " " to ESD-211 PG&E 9/30/77
- Add information to process sheets.





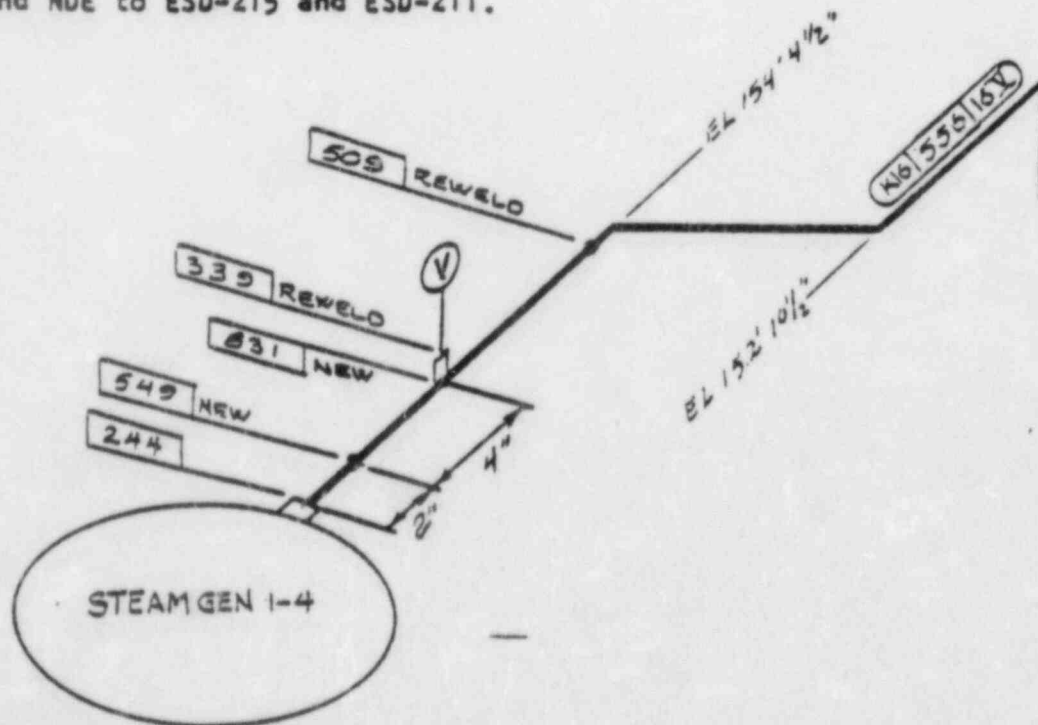
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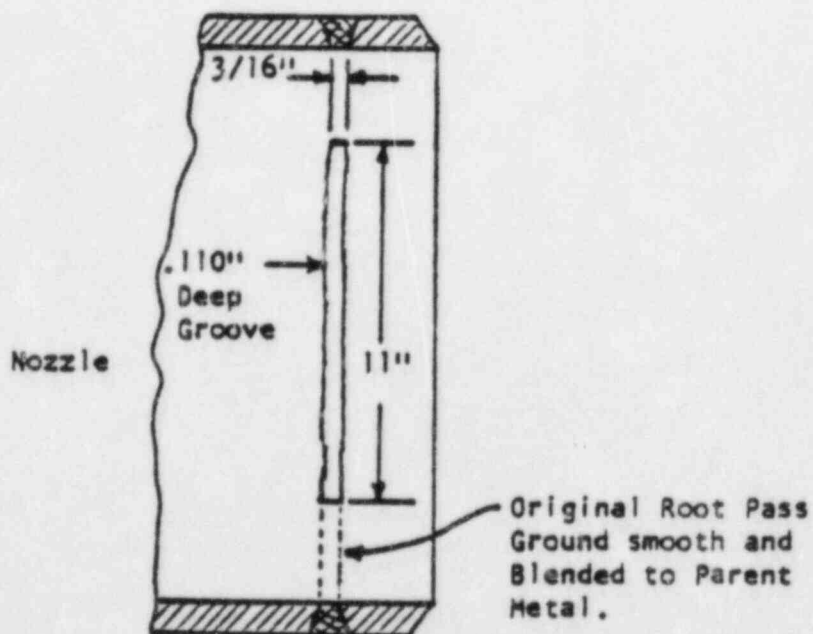
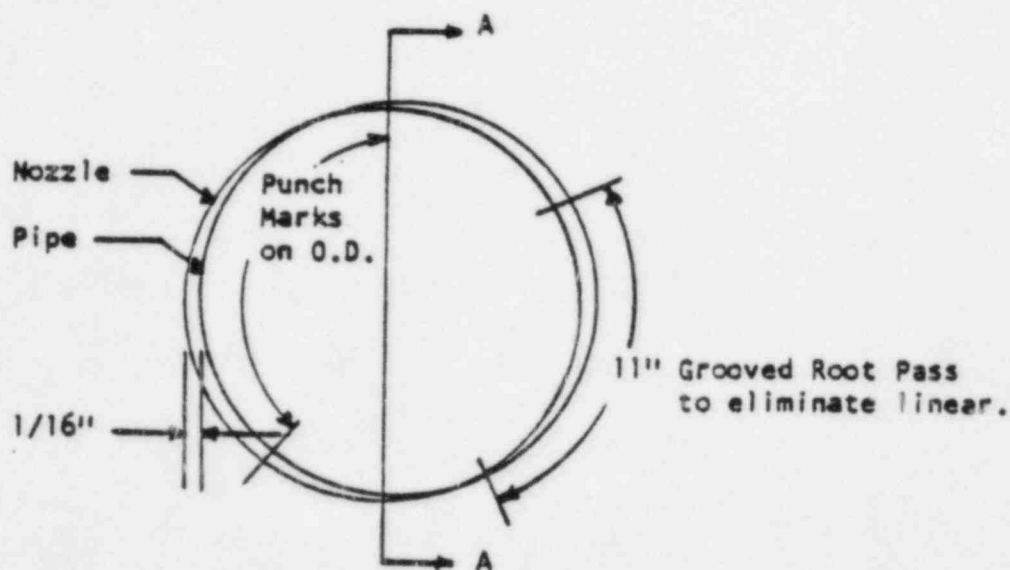
SKETCH #1

3X ACTUAL SIZE

# ALTERNATE INSTRUCTIONS

1. Cut out F.W. 509 using air-arc and taking necessary steps to maintain the ability to bevel the 45° all side of the weld to 20°± 2½° and a 3/32"± 1/16" land. Also, cut out F.W. 339 shown on Iso. 3-246.
2. Requisition: 8½" of 16" Pipe, Schedule 80 A106 Gr B and a 3/4" 3000# S.W. Half Coupling or soc-o-let A105.
3. Install the 3/4" half coupling or soc-o-let four (4) inches from the end of the 16" pipe.
4. Clean and Fit to ESD-215 and ESD-220.
5. Weld out with procedure 92/93 (F.W. 831).
6. Inspect and NDE to ESD-215 and ESD-211.
7. Upon completion of F.W. 244 I.D. inspection and P.G.&E.'s direction, fit-up 16", Schedule 80 pipe and proceed with Step 7 of attached D.R. using F.W. 549 and F.W. 509.
8. Reweld F.W. 339, as shown on Iso. 3-246, maintain original line configuration.
9. Clean and Fit-up to ESD-215 and ESD-220.
10. Weld out with procedure 92/93.
11. Inspect and NDE to ESD-215 and ESD-211.





Section A-A

NOTE: Average Wall Thickness = .580" to .600"  
Remaining Wall At Groove = .450"  
Punch Marks On The O.D. = .047" Deep  
(Punch Marks are directly on the fusion line of F.W. 244 to 1/8" inside the fusion line .047" deep.)

INTEROFFICE CORRESPONDENCE

ATTACHMENT #1  
DR-3453 Rev. 1

TO J. P. RUNYAN, Q.A./Q.C. MANAGER

DATE AUGUST 31, 1977

FROM D. R. GESKE, NDE SUPERVISOR

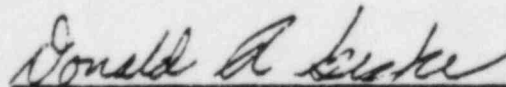
SUBJECT LIQUID PENETRANT EXAMINATION, F.W. 244, INSIDE DIAMETER

On August 31, 1977, F.W. 244, Line K16-556, Isometric 1-03-500146, Feedwater piping to Steam Generator 1-4 Nozzle, was liquid penetrant examined on the inside diameter.

The following indications were noted:

1. 2" from top center of nozzle, a small faint transverse linear-nonrelevant due to weld geometry.
2. 7" from top center of nozzle, heavy 1" drop through with roll over.
3. 9" from top center of nozzle, heavy 2" drop through with roll over.
4. 37" from top center of nozzle, heavy 1½" drop through with roll over.
5. 39" from top center of nozzle, heavy 1/2" drop through with wire protrusion.
6. Approximately 1/16" mismatch exists from 7:00 to 9:00 facing the nozzle.

We recommend that the root pass be ground and polished to blend with base metal surfaces. Particular care should be taken to avoid the removal of base material. Then re-examine the I.D. with liquid penetrant, if acceptable, proceed to "Recommended Disposition", Step 7 of D.R. 3453, Revision 1.



Donald R. Geske  
NDE Supervisor

DRG/js



DR 3453

SCHEDULE TO CUT, INSPECT, RESTORE 1-4 F/W NOZZLE

PLAN: AIR-ARC CUT PIPE 3-2" FROM NOZZLE WELD.

STOP: SHORT OF PIPE I.D. MAINTAINING N<sub>2</sub>

PURGE ON 5/8". REMOVE N<sub>2</sub> PURGE PRESSURE.

FINISH CUT OF PIPE WITH THIS GRINDING DISK.

PULL FLOWING LINE BACK FROM NOZZLE 3 1/2"

TO 4". INSERT PURGE DAM INTO THE 5/8"

NOZZLE TO HOLD POSITIVE PRESSURE ONLY.

(PURGE DAM WILL NOT HOLD PRESSURE)

CONTINUE WORK BY HOLDING LINE AWAY

FROM CUT A MINIMUM OF 3 1/2".

CLEAN I.D. OF NOZZLE WELD WITH SOAP AND

WATER UNTIL CLEAN.

PERFORM P.T.

IF P.T. IS ACCEPTABLE, RESTORE WELD FALDS,

ACCOMPLISH FITUP AND MAKE WELD.

ALTERNATE PLAN: IF MINIMUM OF 3 1/2" OPENING

CANNOT BE ACCOMPLISHED, IT WILL BE

NECESSARY TO MAKE SECOND CUT AT 45°

EW, REMOVE 8" "FLIP" AND FOLLOW ABOVE

STEPS.

# TIME REQUIRED:

1. CUT PIPE & PULL LINE E. 2 MEN, 31" 2 MEN = 12 HRS
2. INSTALL PURGE DAM & RESTORE NL BLANKET - 2 M. 2
3. CLEAN I.D. OF WELD AND PREPARE P.T.
  - 2 MEN - 2 HRS - Q.N. T.M. FOR P.T. - 3 HRS
4. RESTORE WELD PREPARE FITUP - 2 MEN - 16 HRS
5. COMPLETE WELD - 3 M. - 12 HRS
6. RADIOGRAPH WELD - N.I. SHIFT (NO TIME LOST)
7. STRESS RELIEVE WELD - 1 MAN - 6 HRS

## ALTERNATE PLAN: 1, 2, 3 SAME AS ABOVE

4. MAKE SECOND CUT AT 450 LLL - 2 MEN - 3 HRS
5. RESTORE WELD PREPARE ON ADDITIONAL WELD -
  - 2 MEN - 16 HRS
6. MAKE SECOND WELD - 3 MEN - 12 HRS

RADIOGRAPHY & S/R CAN BE DONE AT SAME TIME  
AS ABOVE

BASED ON 40 HR WK - 8 HR DAY

START WORK

8-29-77

TTTT

ST SU MU

11

COMPLETE

9-2-77

ALTERNATE

9-12-77

PULLMAN POWER PRODUCTS

EXHIBIT 9 TO  
ATTACHMENT I

EVALUATION OF DISCREPANCY REPORT IN RELATION  
TO REQUIREMENTS OF 10CFR PART 21

D.R. # 3484, to which this page is attached and  
become a part of ( is not, may be ) considered to be reportable  
to the NRC under the requirements of 10CFR Part 21.

PULLMAN POWER PRODUCTS

by [Signature]

Pacific Gas & Electric Company ( does, does not ) consider this  
discrepancy reportable and ( has reported, will not report ) the  
above discrepancy.

This form returned to Pullman Power Products on 10-10-77.  
DATE

PACIFIC GAS & ELECTRIC CO.

by R.D. GILLER / [Signature]

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

## DISCREPANCY REPORT

Page 1 of 3

Rev. 2

D.R. NO. 3484 Rev. 1  
ISO. NO. 500146 Sht. 1 of 5  
UNIT NO. 1  
CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO. 8711 DATE: 10-7-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-554-16"; Piece #540; El. 154'-5"; Area F; Section I

### EXPLANATION OF DISCREPANCY:

Per P.G.&E. request, an internal inspection of F.W. 197 on Steam Generator 1-1 is to be performed. The line entry will be made in the manner described below.

- ⚠ Inspection conducted October 11, 1977, required by Step #9 of the Recommended Disposition, revealed conditions requiring rework. (See Attachment #3, Report of Findings.)
- ⚠ Inspection conducted October 13, 1977, required by Attachment #3 revealed further rework is required. (See Attachment #4, Report of Rework.)

Attachment #1 (Sketch)

Attachment #2 (Sketch)

⚠ Attachment #3 (Report of Findings)

⚠ Attachment #4 (Report of Rework)

### RECOMMENDED DISPOSITION:

1. Remove the 2", 3000# coupling at F.W. 827 from the pipe using air-arc and grinder (do not air-arc through). Clean and NDE removal area to ESD-220 and ESD-211 on F.W. 197 side of hole in pipe. Close hole with fire retardant material (Reference Attachment #1).
2. Circumscribe the pipe using wrap around at the edge of hole in pipe nearest F.W. 197. This should be approximately 1 5/8" from the center of F.W. 197 (Reference Attachment #1).
3. Prick punch the scribe line.
4. Using air-arc, cut the pipe all around and apply a bevel of not more than 30° from O.D. of pipe to within not less than 1/8" of the I.D. of pipe directly below

Approved By: M.W.K. Field Q.A. Manager *[Signature]* Date 10/10/77 Customer *[Signature]* MUR H-3590 Date 10-10-77 (Con't on Page 2)

FINAL DISPOSITION: ☒ In Accordance With Above

☒ Other (Explanation and approval required)

Work Completed Insp. *[Signature]* Date: 11-2-77

Work Completed Insp. *[Signature]* Date: 11-2-77

### EXPLANATION (IF NECESSARY):

Revision 1 - Adds Attachment #3 (Report of Inspection Findings). P.G.&E. will indicate rework released on Attachment #3 under Recommended Rework. See ⚠ for other changes.

Revision 2 - See page 3 . . . (AND ATT: #4)

M.W.K. Field Q.A. Manager *[Signature]* Date 10/10/77 Customer *[Signature]* MUR H-3590-R1 Date 10-13-77

M.W.K. Field Q.A. Manager *[Signature]* Date 10/14/77 Customer *[Signature]* MUR H-3590-R2 Date 10-17-77

STEPS TO PREVENT RECURRENCE ☒ Not Applicable

Field Q.A. Manager *[Signature]*

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other



# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

Page 2 of 3

## DISCREPANCY REPORT


D.R. NO. 3484 Rev. 1  
ISO. NO. 500146 Sheet 1 of 5  
UNIT NO. 1  
CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 10-7-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-554-16"; Piece #540; El. 154'-5"; Area F; Section I

### RECOMMENDED DISPOSITION:

scribe line (do not cut through the pipe with air-arc). Using a 1/8" blade, grind through to I.D. of pipe.

5. Locate center of F.W. 500. Using wrap around, circumscribe weld center.
6. Prick punch scribe line.
7. Cut out F.W. 500 using air-arc maintaining a maximum bevel of 30° on all side to within not less than 1/8" of the I.D. of pipe (do not cut through with air-arc). Using a 1/8" blade, grind through to I.D. of pipe.
8. Install dam in nozzle of Steam Generator 1-1 past F.W. 197.
9. Clean and inspect F.W. 197 internally as directed by P.G.&E. Report findings of inspection before proceeding with Step #10.  Proceed with rework requirements as directed by P.G.&E. on Attachment #3.
10. Upon completion of inspection, remove dam grind bevels to facilitate fit-up as needed. *AFTER GRINDING IS COMPLETE. JAD*
11. Install new pup, fit-up and inspect to ESD-215 and ESD-220.
12. Weld out F.W. 551 and F.W. 500, as shown on Attachment #2, to process 4/5A0.
13. Inspect and NDE to ESD-215 and ESD-206.
14. Upon acceptance of R.T.'s of F.W. 551 and F.W. 500, clean and fit-up gamma plug, as shown on Attachment #2, to ESD-215 and ESD-220.
15. Weld out to process 7/8 using F.W. 832.
16. Inspect and NDE to ESD-215 and ESD-211.
17. Post weld heat treat to ESD-218 (F.W.'s 551, 500 and 832).
18. Add all information to isometric and process sheets.

Field Q.A. Manager *JAD*

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

PAGE 3 OF 3

## DISCREPANCY REPORT

D.R. NO. 3484 REV. 2  
ISO. NO. 500146, Sht. 1 of 5  
UNIT NO. 1  
CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 10-14-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell, Guest

DISCREPANT ITEM: NOTED

### REVISION 2:

Adds Attachment #4 (Report of Rework). P. G. & E. will indicate rework released on Attachment #4 under Recommended Rework, adds rework to Recommended Disposition as Step 9a.

### RECOMMENDED DISPOSITION:

5 3/8" Linear Removal.

- A. Grind to remove the 5 3/8" linear.
- B. P.T. to ESD 211 to insure complete removal.

Non-metallic Inclusion Removal.

- A. Blend base material to eliminate the two non-metallic inclusions.
- B. U.T. to ESD 244 to determine wall thickness remaining.

Pipe-side Rough Grind & Machining Lines.

- A. Polish the pipe-side counter-bore to eliminate rough grind edges and machining lines.
- B. P.T. to ESD 211 to insure complete removal.
- C. Remove dam prior to Step 4.

Preheat to 250° min. 550° max. interpass; maintain pre-heat until stress relief.

Weld the 5 3/8" Grindout to original I.D. configuration using Procedure 200.

Grind 5 3/8" repair for R.T. while maintaining pre-heat.

M.T. to ESD 207 or Hot P.T. by P. G. & E. D.E.R.

P. G. & E. to indicate N.D.E. to be performed (Step 7).

Weld area of inclusion removal, if needed, to original counter-bore configuration, using Procedure 4/5 AO.

Grind inclusion repair, if Step 8 is used, for R.T. while maintaining pre-heat.

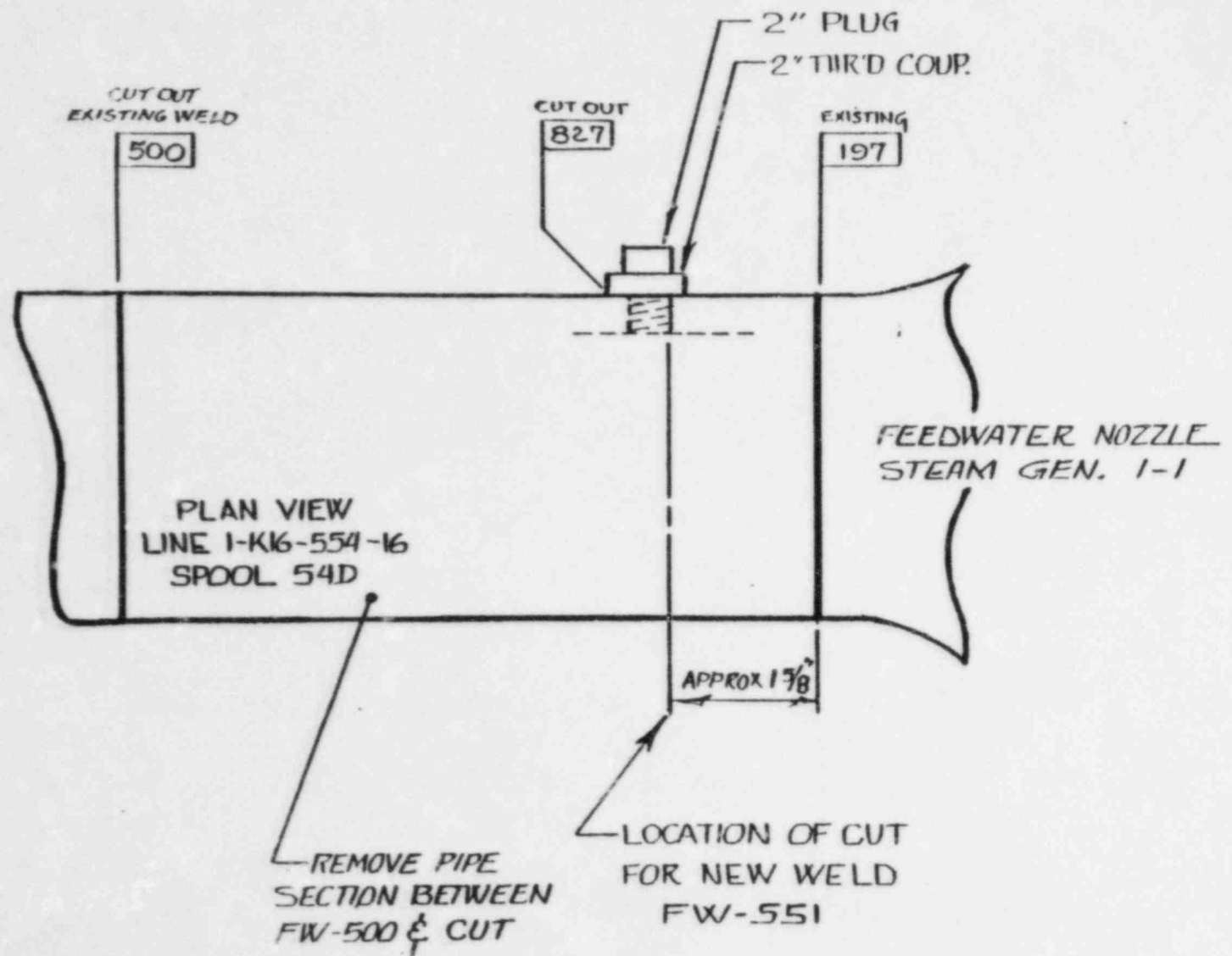
Perform Hot R.T. of repair area per ESD 207.

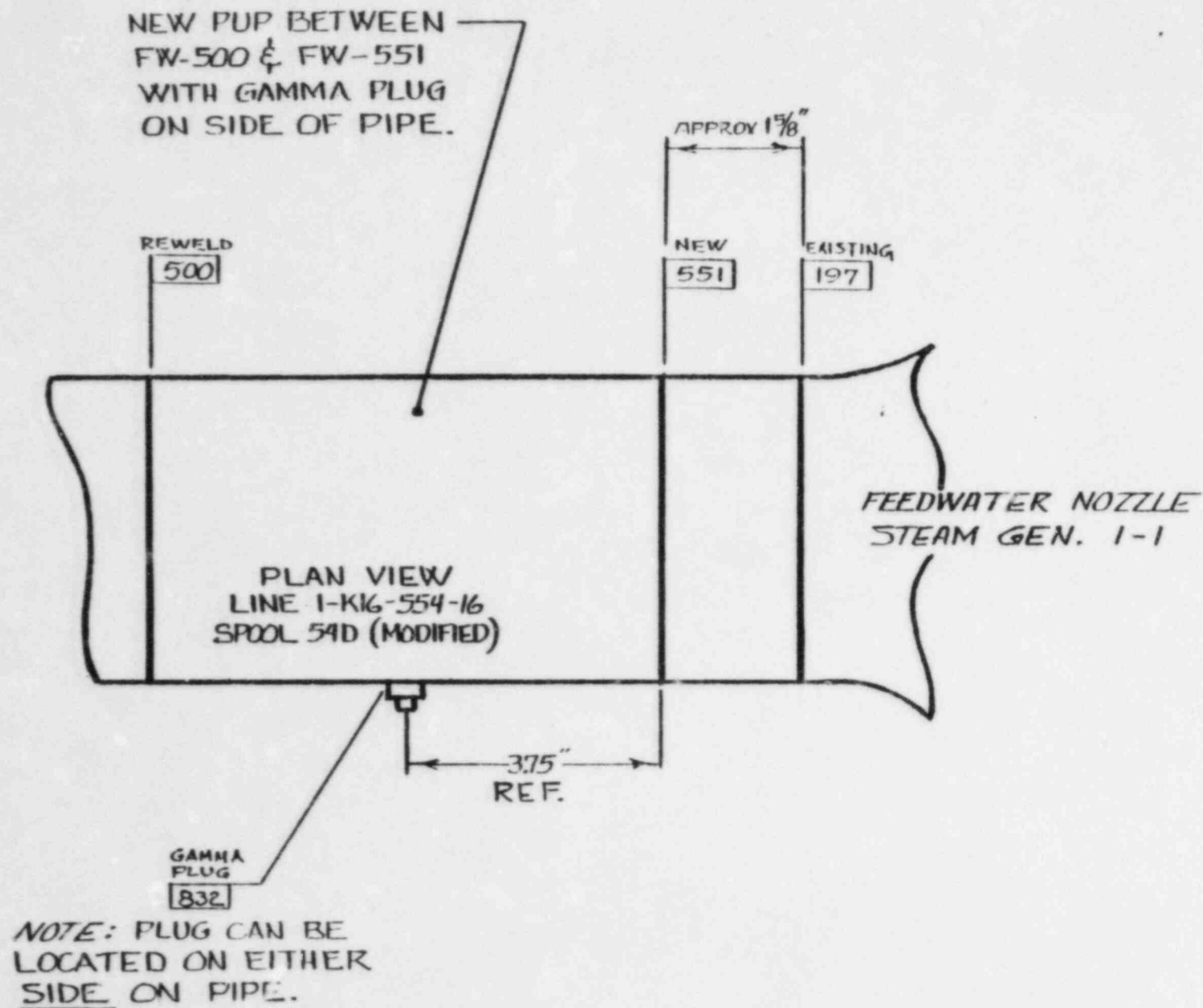
Proceed with Step 10 of the original D.R. 3484 Recommended Disposition.

Field Q.A. Manager

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector ( )

ADD SKETCH IF NECESSARY







## INTEROFFICE CORRESPONDENCE

TO J. P. RUNYAN, FIELD Q.A./Q.C. MANAGER

DATE OCTOBER 12, 1977

FROM D. R. GESKE, N.D.E. SUPERVISOR

SUBJECT LIQUID PENETRANT EXAMINATION, F.W. 197, INSIDE DIAMETER

On October 11, 1977, the inside diameter of F.W. 197 was Liquid Penetrant examined per Step #9 of D.R. 3484. The following indications were noted. All dimensions refer to a clock-wise direction beginning at the top (12 o'clock) of the nozzle.

1. 8" clock-wise: 15" of mis-match with root pass roll over on the nozzle side.
2. 8" clock-wise: 1 3/4" of heavy roll over on the nozzle side and slight roll over on the pipe side. (Previous re-work in this area removed drop-through with voids.)
3. 16" clock-wise: 1" of undercut on the pipe side of the root pass.
4. 18" clock-wise: 1" of undercut on the pipe side of the root pass.
5. 41" clock-wise: 3/16" crack-like linear on the land surface of the counter-bore 3/16" from the root edge, pipe-side.
6. 41 1/2" clock-wise: 1/8" crack-like linear on the land surface of the counter-bore 5/32" from the root edge, pipe-side.
7. 41" clock-wise: 1/16" lack of penetration at the window close area.
8. 46 1/2" clock-wise: 1 1/2" of heavy drop-through with root pass roll over on both sides of the root.
9. The entire pipe-side counter-bore has been rough ground to facilitate fit-up.

### RECOMMENDED REWORK

1. Liquid Penetrant examine to redevelop the noted condition.
2. Grind and polish the root pass. Flush with the base metal from 8" clock-wise for 15", 41" clock-wise at window close, and 46 1/2" clock-wise for 1 1/2". (Indications 1, 2, 3, 4, 7 and 8.)

ATTACHMENT #3

TO: J. P. RUNYAN, Q.A./Q.C. MANAGER

DATE OCTOBER 12, 1977

SUBJECT: LIQUID PENETRANT EXAMINATION, F.W. 197, INSIDE DIAMETER

PAGE NO. 2

3. Grind and polish the land surface to remove the two (2) linear indications in the land area of the counter-bore. (Indications 5 and 6.) *DO NOT REMOVE MORE THAN .010" OF BASE METAL.*

NOTE: If the crack-like linears join and grow, versus reduce in size and remove, suspend grinding and report the findings.

4. Polish the entire pipe-side counter-bore to eliminate rough grind edges. DO NOT remove more than .010" of base metal.
5. Liquid Penetrant examine all grind areas to redevelop indications, i.e. remaining undercut (Indications 3 and 4) and lack of penetration at the window close area (Indication 7).
6. Grind and polish to remove the remaining indications.  
*DO NOT REMOVE MORE THAN .010" OF BASE METAL.*  
NOTE: If the indications grow, versus reduce and remove, suspend grinding and report the findings.
7. Determine the remaining wall thickness using ultrasonic thickness measuring methods and report the findings.
8. Upon completion of rework, proceed with Step #10 of D.R. 3484 or revise D.R. 3484 to indicate further required rework.

*Donald R. Geske*

Donald R. Geske  
N.D.E. Supervisor

DRG/js

*OK  
GTH  
10/12/77*

TO: J. P. RUNYAN, Q.A./Q.C. MANAGER

DATE OCTOBER 12, 1977

SUBJECT: LIQUID PENETRANT EXAMINATION, F.W. 197, INSIDE DIAMETER

PAGE NO. 2

3. Grind and polish the land surface to remove the two (2) linear indications in the land area of the counter-bore. (Indications 5 and 6.) ~~DO NOT REMOVE MORE THAN .010" OF BASE METAL.~~  
*REMOVAL OF .010" DID NOT REMOVE LINEAR. CONTINUOUS BLIND FOR REMOVAL. DO NOT REMOVE BELOW MINIMUM WALL REMAIN.*  
NOTE: If the crack-like linears join and grow, versus reduce in size and remove, suspend grinding and report the findings. *JAY 1072-7*
4. Polish the entire pipe-side counter-bore to eliminate rough grind edges. DO NOT remove more than .010" of base metal.
5. Liquid Penetrant examine all grind areas to redevelop indication, i.e. remaining undercut (Indications 3 and 4) and lack of penetration at the window close area (Indication 7).
6. Grind and polish to remove the remaining indications.  
~~DO NOT REMOVE MORE THAN .010" OF BASE METAL.~~  
NOTE: If the indications grow, versus reduce and remove, suspend grinding and report the findings.
7. Determine the remaining wall thickness using ultrasonic thickness measuring methods and report the findings.
8. Upon completion of rework, proceed with Step #10 of D.R. 3484 or revise D.R. 3484 to indicate further required rework.

DRG/js

*modified 33*  
*OK*  
*gpr 11/12/77*  
Donald R. Geske  
Donald R. Geske  
N.D.E. Supervisor

## INTEROFFICE CORRESPONDENCE

TO J. P. Runyan, Q.A./Q.C. Manager

DATE October 14, 1977

FROM D. R. Geske, N.D.E. Supervisor

SUBJECT Result of Rework, F.W. 197 Inside Diameter

On October 12, 1977, the inside diameter of F.W. 197, Steam Generator 1-1 was reworked per Attachment #3 to D.R. 3484. At the end of shift, the rework had not been completed. The following indications remained.

1. 8" clock-wise from top center of the nozzle, 10" of root pass edge fusion line with slight weld metal remaining above the metal.
2. 41" clock-wise from top center of the nozzle, 3/16" or on the pipe-side counter-bore. The original indication moved 3/32" laterally away from the weld. Tentative interpretation was the open edge of a non-metallic inclusion in the pipe.
3. 41 1/4" clock-wise from top center of the nozzle, 1/8" linear on the pipe side counter-bore. The original indication moved 3/32" laterally away from the weld. Tentative interpretation was the open edge of a non-metallic inclusion in the pipe.

On October 13, 1977, the indications listed above were further reworked. Weld metal was reduced to minimum wall, .566", to eliminate the 10" root pass edge fusion line. In addition, the area of non-metallic inclusion was explored ultrasonically. The following are results of rework.

1. 41" to 41 1/4" clock-wise: two non-metallic inclusions .600" from the O.D. surface, parallel to the surface. The edge of each inclusion was exposed as a result of counter-bore machining. Total surface measures 5/8" x 3/16" with the longest dimension parallel to the weld.
2. 14 1/2" clock-wise: one linear indication 5 3/8" long, in the land surface of the nozzle, 1/16" from the original root pass weld metal. Observation, during weld metal removal and physical measurement, revealed that this indication was hidden by root pass roll over during the first penetrant examination.
3. Pipe side counter-bore surface was not reworked. Rough grind edges and machining lines remain.

ATTACHMENT #4



TO: J. P. Runyan, Q.A./Q.C. Manager

DATE October 14, 1977

SUBJECT: Result of Rework, F.W. 197 Inside Diameter

PAGE NO. 2

Upon reaching minimum wall thickness with indication remaining, rework was suspended and a Q. A. Hold tag applied.

RECOMMENDED REWORK

1. Grind to remove the 5 3/8" linear indication 1/4 1/2" clock-wise from top center of the nozzle. Re-weld the ground-out to original weldment configuration.
2. Grind to remove the two non-metallic inclusions. Measure remaining wall thickness. Weld out to original wall thickness if required and grind to original configuration of the counter-bore.
3. Grind and polish the pipe-side counter-bore to eliminate rough grind edges and machining lines.
4. GRIND AND POLISH ROOT WELD FLUSH WITH PIPE/NOZZLE I.D. *DRG 10-17-77*

Donald R. Reske  
Donald R. Reske  
N.D.E. Supervisor

DRG/jf

*OK  
DRG*

## INTEROFFICE CORRESPONDENCE

TO J. P. RUNYAN, FIELD Q.A./Q.C. MANAGER

DATE OCTOBER 19, 1977

FROM D. R. GESKE, N.D.E. SUPERVISOR

SUBJECT REWORK OF F.W. 197, INSIDE DIAMETER

D.R. 3484, dated October 7, 1977, directed entry into line K16-554-16" approximately 1 5/8" on the pipe-side of F.W. 197. Entry was required to facilitate an internal inspection of the piping to nozzle weldment, identified as F.W. 197.

Entry into the line was completed on October 11, 1977, and the initial inspection was conducted using Liquid Penetrant methods per ESD-211. The inspection revealed conditions requiring rework. These conditions were added to the D.R. by means of Revision 1.

On October 12, 1977, P.G.&E. released the recommended rework noted in Revision 1 to D.R. 3484. The rework was released with a modification "Do not remove more than .010" of base metal". Rework actions were begun. Two (2) linear indications in the pipe-side land surface of the counter-bore were not removed with removal of .010" of base material. P.G.&E. further modified Revision 1 to D.R. 3484 on October 12, 1977, by indicating "Removal of .010" did not remove linears. Continue to blend for removal. Do not remove below minimum wall requirements". At the end of shift, the rework had not been completed.

On October 13, 1977, the remaining indications were further reworked. Pipe and weld wall thickness was reduced to minimum wall (.566") and both linears in the pipe-side counter-bore were explored ultrasonically. Results of these actions were added to D.R. 3484 with Revision 2 on October 14, 1977.

Rework was continued on October 17, 1977, based upon D.R. 3484, Revision 2. Further grinding of weld metal removed all of the existing indications except the pipe-side rough grind and machine lines. Wall thickness at the 5 3/8" linear was reduced to .350" with .520" at the non-metallic inclusions. Revision 2 to D.R. 3484 was then modified by P.G.&E. to include "Grind and polish root weld flush with pipe/nozzle I.D." Rework continued until end of shift.

Liquid Penetrant examination was conducted on October 18, 1977, after preliminary grind and polish of root pass removal and pipe-side counter-bore rough grind edges and machining lines. The examination revealed the following conditions:

1. 17" from top center of the nozzle - 4 1/2" x 1/8" area of a fine network of linears on either side of the weld centerline. These two networks were located in the base metal on the face of each counter-bore.

TO: J. P. RUNYAN, FIELD Q.A./Q.C. MANAGER

DATE OCTOBER 19, 1977

SUBJECT: REWORK OF F.W. 197, INSIDE DIAMETER

PAGE NO. 2

2. 31" from top center of the nozzle - 2 1/2" fine linear on either side of the root pass removal area.

Both conditions noted above were reported by telephone. Further rework was stopped pending review by P.G.&E.

The areas in question were examined by a representative from P.G.&E. Mechanical, P.G.&E. D.E.R. and Westinghouse on October 18, 1977. Instruction from P.G.&E. Mechanical was to remove the indications by grind and polish. Further discussion resulted in instruction to proceed under the existing revision to D.R. 3484. Rationale applied was that the two (2) linears at 31" was part of root pass removal and the network of linears at 17" was part of the rough grind clean-up action.

Rework of the inside surface continued to the end of shift. All noted indications were removed and a preliminary Liquid Penetrant examination verified removal.

Liquid Penetrant examination per ESD-211 was re-applied on October 19, 1977. Examination revealed all previously noted discontinuities were removed and rework was concluded. Ultrasonic thickness measurements of remaining wall indicated average wall in the flush grind areas is .580". Grind areas are as follows:

1. 8" clock-wise from top center of the nozzle a 10" crescent groove .350" of remaining wall.
2. 31" clock-wise from top center of the nozzle a 2 1/2" crescent groove .420" of remaining wall.
3. 41" clock-wise from top center of the nozzle a 2" rounded grind-out .520" of remaining wall.

On October 19, 1977, rework of F.W. 197 inside surface was continued per D.R. 3484, Revision 2, Step #9A, Item 4.

*Donald R. Geske*

D. R. Geske  
N.D.E. Supervisor

DRG/js

Attachment - D.R. 3484, Rev. 2

THE H.W. KELLOGG COMPANY  
A DIVISION OF PULLMAN INCORPORATED

LIQUID PENETRANT EXAMINATION RECORD

ISO/DWG NUMBER 500146A (DR 3484, REV 2) DATE 10-19-77  
JOINT NUMBER FW-197 (INSIDE SURFACE AFTER GRIND OUT) "F" SHEET N/A  
MATERIAL 16" C.S. TO STEAM GENERATOR 1-1 NOZZLE JOB NO. 7177  
EXAMINATION PROCEDURE KPT- 8711 ESD- 211  
ACCEPTANCE STANDARDS ASME SECTION I  
LIQUID PENETRANT BRAND MAGNAFLUX DYE CHECK  
LIQUID PENETRANT BATCH NUMBER CLEANER 6J004 PENETRANT 2L095 DEVELOPER 5M113

☒ ACCEPTED

☐ REJECTED

Donald R. Burke  
SIGNATURE OF INSPECTOR

MAGNETIC PARTICLE EXAMINATION RECORD

DWG NUMBER \_\_\_\_\_ DATE \_\_\_\_\_  
JOINT NUMBER FW- "F" SHEET N/A  
MATERIAL CARBON STEEL JOB NO. 7177  
EXAMINATION PROCEDURE KMT- 8711 ESD-  
ACCEPTANCE STANDARDS ASME  
MAGNETIC PARTICLE POWDER BRAND MAGNAFLUX  
MAGNETIC PARTICLE POWDER COLOR RED (8A)  
COLOR OF COMPONENT METALLIC  
INDICATIONS  
Linear  
Round  
Aligned

☐ ACCEPTED

☐ REJECTED





THE H.W. KELLOGG COMPANY  
A DIVISION OF PULLMAN INCORPORATED

LIQUID PENETRANT EXAMINATION RECORD

ISO/DWG NUMBER \_\_\_\_\_ DATE \_\_\_\_\_  
JOINT NUMBER FW- "F" SHEET N/A  
MATERIAL \_\_\_\_\_ JOB NO. 7177  
EXAMINATION PROCEDURE KPT- 8711 ESD-  
ACCEPTANCE STANDARDS ASME  
LIQUID PENETRANT BRAND MAGNAFLUX DYE CHECK  
LIQUID PENETRANT BATCH NUMBER CLEANER PENETRANT DEVELOPER  
☐ ACCEPTED ☐ REJECTED  
SIGNATURE OF INSPECTOR \_\_\_\_\_

MAGNETIC PARTICLE EXAMINATION RECORD

DWG NUMBER 500146A (DR 3484, REV 2) DATE 10-19-77  
JOINT NUMBER FW-197 (INSIDE SURFACE AFTER WELD-OUT) (300°F) "F" SHEET N/A  
MATERIAL CARBON STEEL JOB NO. 7177  
EXAMINATION PROCEDURE KMT- 8711 ESD- 247 10-28-77  
ACCEPTANCE STANDARDS ASME SECTION I  
MAGNETIC PARTICLE POWDER BRAND MAGNAFLUX  
MAGNETIC PARTICLE POWDER COLOR RED (8A)  
COLOR OF COMPONENT METALLIC  
INDICATIONS  
Linear  
Round  
Aligned  
NOTE: COIL METHOD WAS USED.  
10-28-77



# Pullman Power Products

EXHIBIT 10 TO  
ATTACHMENT I

KN

10 CFR-21  
(IS) or (IS NOT)  
ATTACHED

## DISCREPANCY REPORT

D.R. NO. 4662  
ISO. NO. 500146  
UNIT NO. I  
CODE NO. 8

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 11-8-82  
PROJECT: Diablo Canyon JOE NO.: 7177 INSPECTOR: E. Hudson/MacCrae

DISCREPANT ITEM: ESD-246 and ESD-247 - Magnetic Particle Procedure/ Dry/Continuous Coil Method  
not per Spec 8711

### EXPLANATION OF DISCREPANCY:

Internal Audit #101, A.A.R. #1, identified that there was no Procedure Qualification Record for ESD-246 and ESD-247.

Subsequent investigation has revealed that ESD-247 was used to MT examine the inside diameter of FW #197, Iso #500146, Steam Generator 1-1, DR #3484, on two occasions (see attached examination records) ESD-247 is listed on DR #3453 as well, under Recommended Disposition and is suspected to have been used (there is no examination record with the D.R.) to MT examine the inside diameter of FW #244, Iso #500146, Steam Generator 1-4. The welds in question are on lines K16-554-16 IV and K16-556-16 IV, both Seismic Class 1 code.

The use of ESD-247 on Seismic Class 1 pipe without a Procedure Qualification Record is a nonconformance to Contract Specification 8711 Section 3 Table I and Notes to Table I, Section Paragraph 3.23, Section 5 Paragraph 1.2, Section 2 Paragraph 2.1 and Section 3 Paragraph 1.4.

--CONTINUED ON PAGE 2 OF DISCREPANCY REPORT--

### RECOMMENDED DISPOSITION:

INDICATE APPROVAL BY CIRCLING THE APPROPRIATE "RECOMMENDED DISPOSITION"

- ① Accept-as-is all work examined by ESD-247
- ② File a copy of this D.R. in the following locations:
  - A. Document package for Iso #500146 FW 197 and original DR 3484
  - B. Document package for Iso #500146 FW 244 and original D.R. 3453
  - C. Originals of ESD 246 and 247

OR

☒ P.G.&E. to disposition

PG&E G.C.  
QUALITY CONTROL

REVIEWED

T.E. Quinn

DATE

12/9/83

Approved By P.P.P. Field C.A. Mgr. [Signature] Date 11/29/82 Customer [Signature] Date 2/9/83

NAL DISPOSITION: ☒ In Accordance With Above #1 & #2

☐ Other (explanation and approval required)

Work Completed Inset: [Signature] Date: 1/14/83

Work Completed Inset: \_\_\_\_\_ Date: \_\_\_\_\_

EXPLANATION (IF NECESSARY):

P.P.P. Field C.A. Manager \_\_\_\_\_ Date \_\_\_\_\_ Customer \_\_\_\_\_ Date \_\_\_\_\_

TO PREVENT RECURRENCE ☒ Not Applicable

A/QC Manager shall assure that N.D.E. procedures have the necessary Procedure Qualification records performed and included as part of the procedure prior to submitting to P.G.&E. for approval.

Field C.A. Manager [Signature] 1/29/83

DISTRIBUTION: ☒ Master C.A. File ☒ Auth. Inset ☒ Engineering Dept. ☐ Other \_\_\_\_\_



# Pullman Power Products

10 CFR-21  
(IS) or (IS NOT)  
ATTACHED

## DISCREPANCY REPORT

D.R. NO. \_\_\_\_\_  
ISO. NO. 500146  
UNIT NO. 1  
CODE NO. 7

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 11-8-82  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: W. J. [unclear]

Generation 1-4. The wellbore on lines K16-554-16IV and K16-556-16IV, both Sermie Class 1 wells. The well ESD 247 on Sermie Class 1 pipe within a flowline qualification record as a nonconformance to Contract Specification 5711 Section 3 Table I and Notes to Table I, Section 4.3.23, Section 5.1.2 Section 2.2.01 and Section 3.14.

THE H.W. KELLOGG COMPANY  
A DIVISION OF PULLMAN INCORPORATED

LIQUID PENETRANT EXAMINATION RECORD

ISO/DWG NUMBER \_\_\_\_\_ DATE \_\_\_\_\_  
JOINT NUMBER FW- "F" SHEET N/A  
MATERIAL \_\_\_\_\_ JOB NO. 7177  
EXAMINATION PROCEDURE KPT- 8711 ESD-  
ACCEPTANCE STANDARDS ASME  
LIQUID PENETRANT BRAND MAGNAFLUX DYE CHECK  
LIQUID PENETRANT BATCH NUMBER CLEANER PENETRANT DEVELOPER



ACCEPTED



REJECTED

SIGNATURE OF INSPECTOR

MAGNETIC PARTICLE EXAMINATION RECORD

DATE 10-19-77  
/DWG NUMBER 500146A (DR 3484, REV 2) "F" SHEET N/A  
JOINT NUMBER FW-197 (INSIDE SURFACE AFTER WELD-OUT) (300°F) JOB NO. 7177  
MATERIAL CARBON STEEL  
EXAMINATION PROCEDURE KMT- 8711 ESD- 247 10-28-77  
ACCEPTANCE STANDARDS ASME SECTION I  
MAGNETIC PARTICLE POWDER BRAND MAGNAFLUX  
MAGNETIC PARTICLE POWDER COLOR RED (8A)  
COLOR OF COMPONENT METALLIC

INDICATIONS

☒ Linear  
☐ Round  
☐ Aligned

NOTE: COIL METHOD WAS USED

10-28-77



ACCEPTED



REJECTED

Donald R. Cooke

SIGNATURE OF INSPECTOR



THE W.W. KELLOGG COMPANY  
A DIVISION OF PULLMAN INCORPORATED

LIQUID PENETRANT EXAMINATION RECORD

DATE \_\_\_\_\_  
O/T/G NUMBER \_\_\_\_\_ "F" SHEET N/A  
JOINT NUMBER FW- JOB NO. 7177  
MATERIAL \_\_\_\_\_  
EXAMINATION PROCEDURE KPT- 8711 ESD-  
ACCEPTANCE STANDARDS ASME  
LIQUID PENETRANT BRAND MAGNAFLUX DYE CHECK  
LIQUID PENETRANT BATCH NUMBER CLEANER PENETRANT DEVELOPER  
☐ ACCEPTED ☐ REJECTED  
SIGNATURE OF INSPECTOR \_\_\_\_\_

MAGNETIC PARTICLE EXAMINATION RECORD

DATE 10-20-77  
SO/DWG NUMBER 500146A (DR 3484) "F" SHEET N/A  
JOINT NUMBER FW- 197 (INSIDE SURFACE AFTER WELD) JOB NO. 7177  
MATERIAL CARBON STEEL  
EXAMINATION PROCEDURE KMT- 8711 ESD- 247  
ACCEPTANCE STANDARDS ASME SECTION I  
MAGNETIC PARTICLE POWDER BRAND MAGNAFLUX  
MAGNETIC PARTICLE POWDER COLOR RED (8A)  
COLOR OF COMPONENT METALLIC  
INDICATIONS  
Linear  
Round  
Aligned  
☒ ACCEPTED ☐ REJECTED  
SIGNATURE OF INSPECTOR Donald R. Burke



# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

Page 1 of 3

D.R. NO. 3453 Rev. 1 Rev.  
ISO. NO. 500146 Shr. 4 of 5  
UNIT NO. 1  
CODE NO. N/A

## DISCREPANCY REPORT

CUSTOMER: Pacific Gas & Electric SPEC. NO: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16"; Spool Piece 500; El. 154'-4 1/2"; Area G; Sect. 1

EXPLANATION OF DISCREPANCY: Reference P.G.&E. Drawing 500146, Rev. 98

Per P.G.&E.'s request, an internal inspection of F.W. 244 on Steam Generator 1-4 Feedwater Nozzle will be performed. The line entry will be made in the following manner, see Recommended Disposition.

FOR INFORMATION ONLY

### RECOMMENDED DISPOSITION:

1. Measure 2" from F.W. 244 center upstream on pipe (Spool Piece 500) circumscribe the pipe using a wrap around, then prick punch the scribe line.
2. Using air-arc, cut the pipe all around maintaining less than a 40° total bevel to within approximately 1/8" from the I.D. (See Sketch #1).
3. Using a thin blade grinder, cut through the center of the bevel, then dress up both bevels (See Sketch #1).
4. Jack pipe away from cut on upstream side per Superintendent's instruction. Install a dam in nozzle beyond F.W. 244 to prevent air from entering Steam Generator.
5. Clean and inspect F.W. 244 internally as directed by P.G.&E. (Con't on Page 2)

Approved By: M.W.K. Field Q.A. Mgr. J.P. Runyan Date 8/26/77 Customer R.D. Etler Date 8/29/77 *MUR 3553*

FINAL DISPOSITION: ☒ In Accordance With Above

☒ Other (explanation and approval required)

Work Completed Insp. AD Burke Date: 11-2-77

Work Completed Insp. AD Burke Date: 11-2-77

### EXPLANATION (IF NECESSARY):

Revision 1 - Delete Step 6 in its entirety (access was achieved). Refer to Attachment #1 for results of initial inspection and recommended disposition.

Revision 2 - See Page 3 of 3 and Attachment #3.

Rev #2 J.P. Runyan 9/7/77

Rev #1 J.P. Runyan 8/31/77

M.W.K. Field Q.A. Manager J.P. Runyan Date 8/31/77 Customer R.D. Etler Date 8/29/77 *DEVIATION 254*

STEPS TO PREVENT RECURRENCE ☒ Not Applicable J.P.

Field Q.A. Manager

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector

ATTACH SKETCH IF NECESSARY

# THE M.W. KELLOGG COMPANY

A DIVISION OF PULLMAN INCORPORATED

## DISCREPANCY REPORT

Page 2 of 3

D.R. NO. 3453 Rev. 1 REL  
ISO. NO. 500146 Sht. 4 of 5  
UNIT NO. 1  
CODE NO. N/A

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16"; Spool Piece 500; El. 154'-4 1/2"; Area G; Sect. 1

### RECOMMENDED DISPOSITION: (CON'T)

6. If access to F.W. 244 I.D. cannot be achieved, proceed with alternate instructions (See Sketch #2).
7. Upon completion of inspection, remove dam, grind bevel to facilitate fit-up as needed, clean and fit to ESD-215 and ESD-220.
8. Weld out using F.W. 549 and procedure 4/5.
9. Inspect and NDE to ESD-215 and ESD-206.
10. Add all information to process sheet and isometrics.

FOR INFORMATION ONLY

DISTRIBUTION: ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other \_\_\_\_\_  
☒ Customer ☐ Receiving ☐ Field Inspector ( \_\_\_\_\_ )

Page 3 of 3.

A DIVISION OF PULLMAN INCORPORATED

### DISCREPANCY REPORT

D.A. NO. 3453 REV 2  
ISO. NO. 500146 Shr. 4 of 5  
UNIT NO. 1  
CODE NO. 4/A

CUSTOMER: Pacific Gas & Electric SPEC. NO.: 8711 DATE: 8-26-77  
PROJECT: Diablo Canyon JOB NO.: 7177 INSPECTOR: Runyan/Scannell/Guest

DISCREPANT ITEM: Line K16-556-16"; Spool Piece 500; El. 154'-4½"; Area G; Sect. 1

D.R. 3453, Revision 2

EXPLANATION OF DISCREPANCY:

During the performance of grinding per D.R. 3453, Revision 1, Step #5, four (4) indications of heavy drop through with roll over were removed along with all of the existing root penetration. Re-inspection revealed a linear indication .11" long on the fusion line to the Feedwater Nozzle. Further grinding, to a maximum depth of .110", removed the linear indication (see Attachment #3).

the Feedwater Nozzle. Further grinding, to a maximum depth of 0.005 inches, is required to obtain the proper indication (see Attachment #3).

RECOMMENDED DISPOSITION:

Add the following to Step #5 and delete original Steps #7, 8, 9 and 10. Perform the following repair:

- A. Grind the bevels to facilitate Fit-Up of F.W. 549.  
B. M.T. I.D. of F.W. 244 using ESD-247 coil method; and clean grooved root pass at F.W. 244 per ESD-215.  
C. Remove the dam from the nozzle at F.W. 244.  
D. Preheat F.W. 244 to a minimum of 250°F; interpass not to exceed 550°F maximum. Maintain this preheat until stress relief begins.  
E. Weld to procedure 200 (F.W. 244 R1), using E70S2 wire with the GTAW process, to conform to blended configuration of the balance of the weld. I.D. ONLY  
F. Grind and polish the added weld metal to blend with base metal surfaces.  
G. Inspect and NDE to ESD-215 and PLEASE DER TO PERFORM L.P. AT PREHEAT-TEMP.  
FURNISH MWK WITH COPY OF PQC CERT.

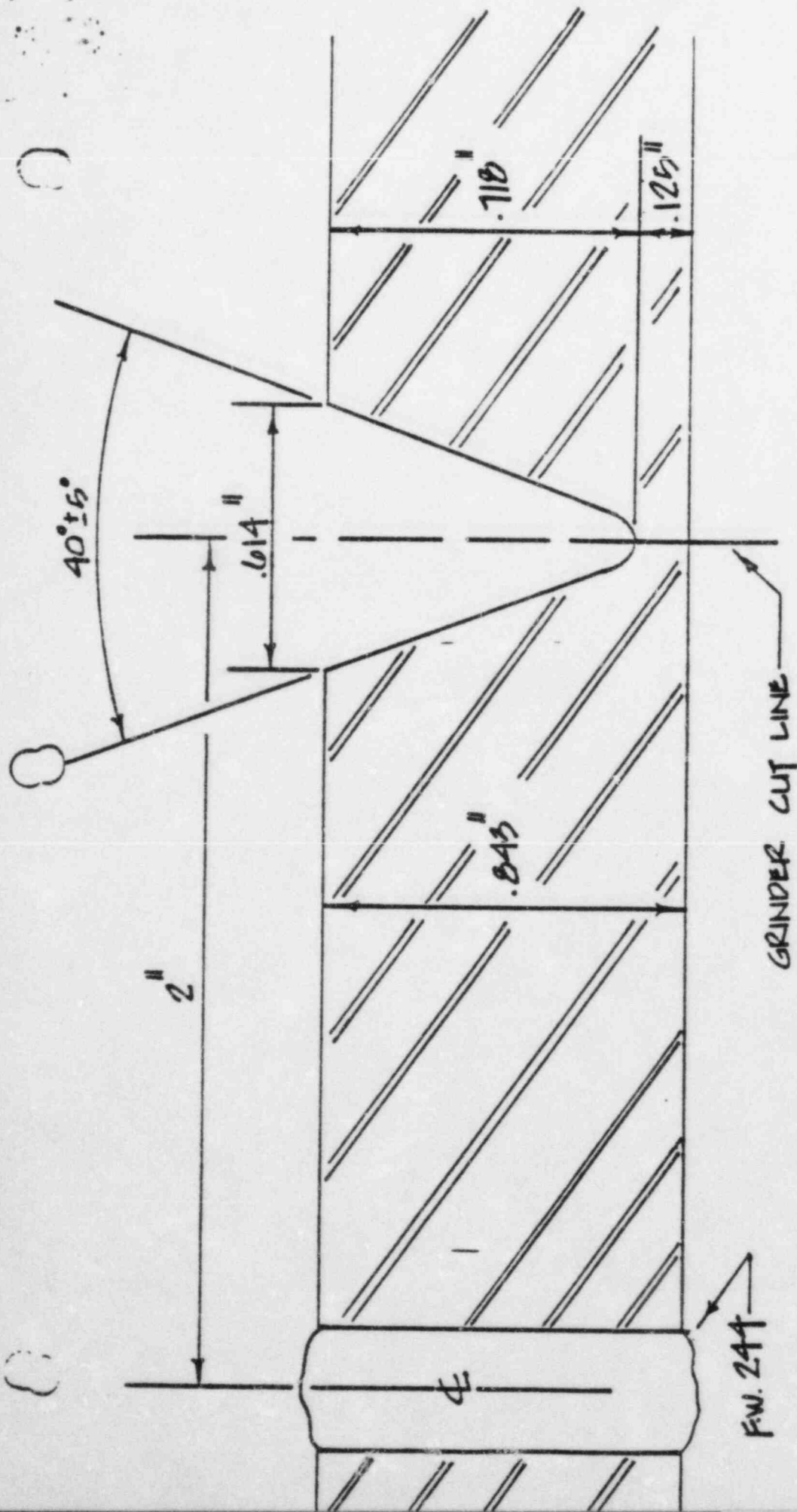
NOTE: Proceed with D.R. 3456, and X-ray weld, hot, for information only.

7. Upon completion of inspection, remove dams from F.W. 549 pipe side, clean and fit F.W. 549 to ESD-215 and ESD-220.
8. Weld out using F.W. 549 with procedure 4/5A.D.
9. Grind F.W. 549 for X-ray.
10. Inspect and NDE F.W. 549 to ESD-215 & ~~ALL NDE TO BE COMPLETED BY 10/30/77~~  
~~10/30/77 WITH COPY OF 20000 CORRECTED BY 10/30/77~~

NOTE: X-ray weld hot for information only.

11. Stress relieve at F.W. 244 R1 and F.W. 549 1100°F to 1150°F per ESD-218.
12. NDE F.W. 244 R1 and F.W. 549 to ESD-207.
13. " " " " to ESD-211 *add 9/30/77*
14. Add information to process sheets.

**DISTRIBUTION:** ☒ Master Q.A. File ☒ Auth. Insp. ☒ Engineering Dept. ☐ Other  
☒ Customer ☐ Receiving ☐ Field Inspector



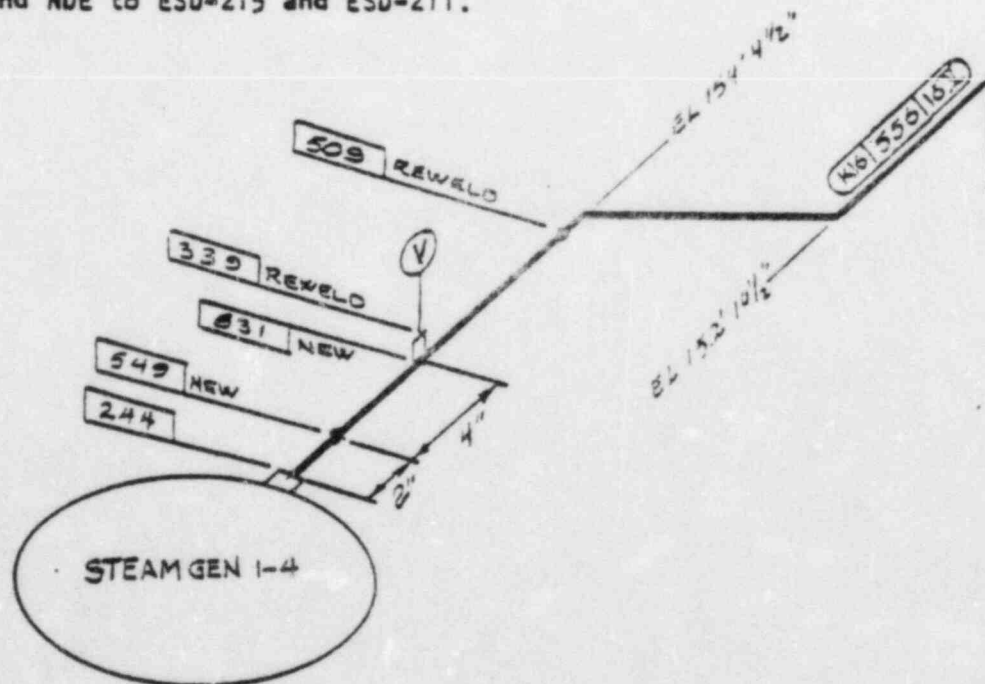
#  
DR-3453  
SKETCH #1

3X ACTUAL SIZE

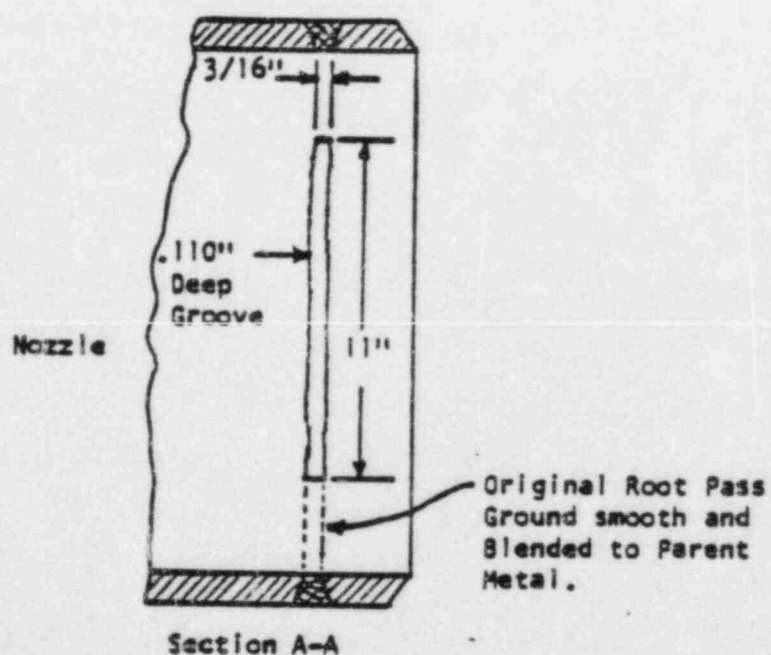
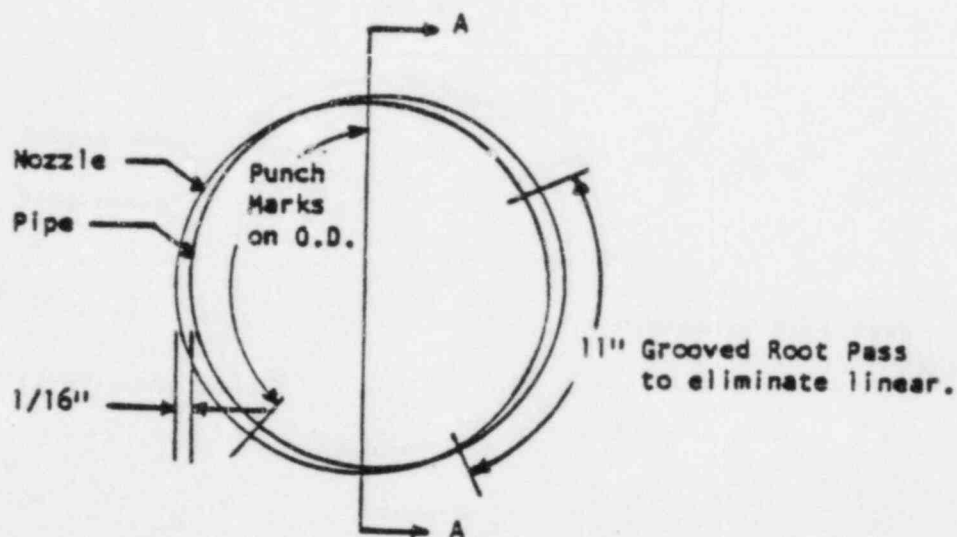


# ALTERNATE INSTRUCTIONS

1. Cut out F.W. 509 using air-arc and taking necessary steps to maintain the ability to bevel the  $45^\circ$  all side of the weld to  $20^\circ \pm 2\frac{1}{2}^\circ$  and a  $\frac{3}{32}'' \pm \frac{1}{16}''$  land. Also, cut out F.W. 339 shown on Iso. 3-246.
2. Requisition:  $8\frac{5}{8}''$  of 16" Pipe, Schedule 80 A106 Gr B and a  $\frac{3}{4}''$  3000# S.W. Half Coupling or soc-o-let A105.
3. Install the  $\frac{3}{4}''$  half coupling or soc-o-let four (4) inches from the end of the 16" pipe.
4. Clean and Fit to ESD-215 and ESD-220.
5. Weld out with procedure 92/93 (F.W. 831).
6. Inspect and NDE to ESD-215 and ESD-211.
7. Upon completion of F.W. 244 I.D. inspection and P.G.&E.'s direction, fit-up 16", Schedule 80 pipe and proceed with Step 7 of attached D.R. using F.W. 549 and F.W. 509.
8. Reweld F.W. 339, as shown on Iso. 3-246, maintain original line configuration.
9. Clean and Fit-up to ESD-215 and ESD-220.
10. Weld out with procedure 92/93.
11. Inspect and NDE to ESD-215 and ESD-211.



Findings at F.W. 244



NOTE: Average Wall Thickness = .580" to .600"  
Remaining Wall At Groove = .450"  
Punch Marks On The O.D. = .047" Deep  
(Punch Marks are directly on the fusion line of F.W. 244 to 1/8" inside the fusion line .047" deep.)

INTEROFFICE CORRESPONDENCE

ATTACHMENT #1  
DR-3453 Rev. 1

TO J. P. RUNYAN, Q.A./Q.C. MANAGER

DATE AUGUST 31, 1977

FROM D. R. GESKE, NDE SUPERVISOR

SUBJECT LIQUID PENETRANT EXAMINATION, F.W. 244, INSIDE DIAMETER

On August 31, 1977, F.W. 244, Line K16-556, Isometric 1-03-500146, Feedwater piping to Steam Generator 1-4 Nozzle, was liquid penetrant examined on the inside diameter.

The following indications were noted:

1. 2" from top center of nozzle, a small faint transverse linear-nonrelevant due to weld geometry.
2. 7" from top center of nozzle, heavy 1" drop through with roll over.
3. 9" from top center of nozzle, heavy 2" drop through with roll over.
4. 37" from top center of nozzle, heavy  $1\frac{1}{2}$ " drop through with roll over.
5. 39" from top center of nozzle, heavy  $1\frac{1}{2}$ " drop through with wire protrusion.
6. Approximately  $1/16$ " mismatch exists from 7:00 to 9:00 facing the nozzle.

We recommend that the root pass be ground and polished to blend with base metal surfaces. Particular care should be taken to avoid the removal of base material. Then re-examine the I.D. with liquid penetrant, if acceptable, proceed to "Recommended Disposition", Step 7 of D.R. 3453, Revision 1.



Donald R. Geske  
NDE Supervisor

DRG/js

Response to Internal Audit #101, A.A.R. #1

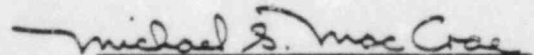
A procedure qualification was performed in order to qualify ESD-247 as well as ESD-246 (ref. Internal Audit #101, A.A.R. #1).

As both ESD-246 and 247 have been withdrawn from use and the equipment listed in both procedures is no longer available for use, the equipment used for these PQR's was the Magnaflux Model P-90 for the coil technique only. All other factors are the same as specified in the procedures.

As the Magnaflux Model P-90 puts <sup>out</sup> less amperage than the equipment listed in ESD-246 and ESD-247, and acceptable results were obtained (i.e; I.D. and O.D. surface and I.D. subsurface indications were located), the demonstration of this technique is deemed adequate to approve this method employed by the equipment listed in the above procedures.

A record of the procedure qualification demonstration is hereby submitted along with this response. The originals will be filed with the applicable procedures.

In the event that either or both procedures are re-activated in the future, they will be revised to incorporate the Magnaflux Model P-90 into the list of equipment. As both procedures are currently withdrawn from use, no revision is deemed necessary at this time.



Michael S. MacCrae  
MT Level III  
11-9-82

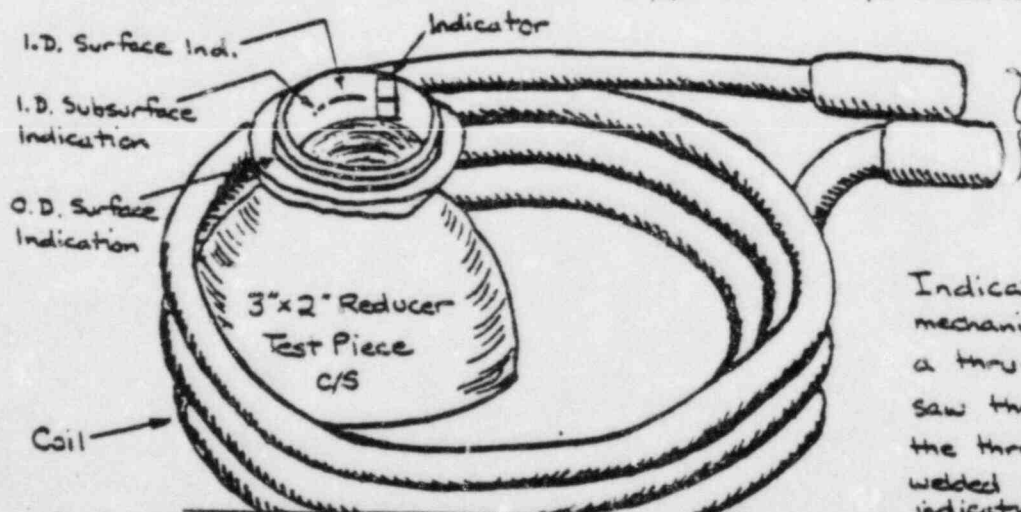


## ENGINEERING DEPARTMENT

ES D-246

MAGNETIC PARTICLE  
PROCEDURE QUALIFICATION RECORDProcedure Qualification No. MT-3

Method Dry Powder Continuous; Half Wave D.C., Coil  
Equipment Manufacture Magnaflux Corp.  
Brand Name Magnaflux  
Model Number P-90  
Magnetic Particles: Brand Magnaflux Type Dry Color 8A-Red  
Magnetization Current Half Wave Rectified D.C.  
Out Put Amperes 1000 Amps.  
Test Piece Position Vertical  
Test Material Carbon Steel  
Type Test Piece 3"x2" Reducer with cut fillet weld at socket end  
Surface Condition Bare Metal  
Pre-Cleaning Method None  
Weather 70° Dry  
Base Metal Color Dark Brown and Shiny Metal  
Contrast of Particles to Base Metal Good  
Method of Particle Application Powder Bulb  
Method of Removing Excess Particles Blowing  
Demagnetization None  
Prod. Spacing and Amperage: 3 Coil Loops of 5 x Pipe Diameter with 1000 Amps.



Coil plugged into  
Magnaflux Model P-  
Half-Wave Rectified  
DC Current, 110 Volt

Indications are induced  
mechanical defects made by  
a thru-wall cut of a band  
saw then compressed. Part of  
the thru wall cut was back-  
welded to produce sub-surface  
indications.

Evaluation of Procedure Good-Indicator and defect showed well.

Procedure: ☐ Approved ☐ Not Approved

Test Conducted By M. MacCrae 11/9/82

M. S. MacCrae  
MT Level III

PREPARED BY M. MacCrae

DATE OF ISSUE 11/9/82

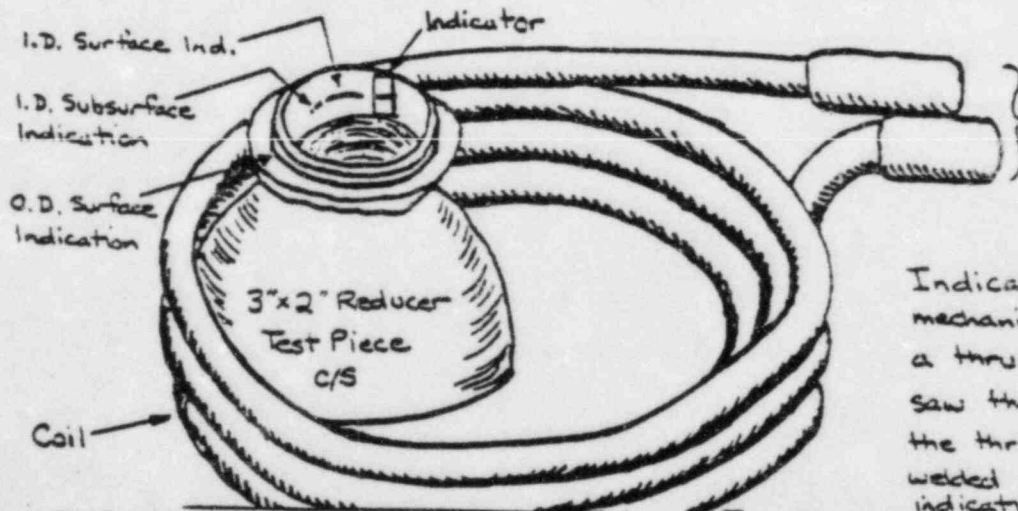
PAGE 1 OF 1

DATE OF REV. \_\_\_\_\_

APPROVED BY \_\_\_\_\_

## ENGINEERING DEPARTMENT

ES D-247

MAGNETIC PARTICLE  
PROCEDURE QUALIFICATION RECORDProcedure Qualification No. MT-3Method Dry Powder Continuous; Half Wave D.C., CoilEquipment Manufacture Magnaflux Corp.Brand Name MagnafluxModel Number P-90Magnetic Particles: Brand Magnaflux Type Dry Color 8A-RedMagnetization Current Half Wave Rectified D.C.Out Put Amperes 1000 Amps.Test Piece Position VerticalTest Material Carbon SteelType Test Piece 3"x2" Reducer with cut fillet weld at socket endSurface Condition Bare MetalPre-Cleaning Method NoneWeather 70° DryBase Metal Color Dark Brown and Shiny MetalContrast of Particles to Base Metal GoodMethod of Particle Application Powder BulbMethod of Removing Excess Particles BlowingDemagnetization NoneProd. Spacing and Amperage: 3 Coil Loops of 5 x Pipe Diameter with 1000 Amps.

Coil plugged into  
Magnaflux Model P-  
Half-Wave Rectified  
DC Current. (110 Volt)

Indications are induced  
mechanical defects made by  
a thru-wall cut of a band  
saw then compressed. Part of  
the thru wall cut was back-  
welded to produce sub-surface  
indications.

Evaluation of Procedure Good-Indicator and defect showed well.Procedure: ☐ Approved ☐ Not ApprovedTest Conducted By M. MacCrae 11/9/82M. S. MacCrae  
MT Level IIIPREPARED BY M. MacCraeDATE OF ISSUE 11/9/82PAGE OF

APPROVED BY \_\_\_\_\_

DATE OF REV. \_\_\_\_\_



THE H. W. KESSLER COMPANY  
A DIVISION OF PULMAN INCORPORATED

## PERSONNEL TESTING RECORD

EXHIBIT II TO  
ATTACHMENT I

NAME D. R. GESKE

SUBJECT	LEVEL	GRADE	DATE	EXAMINED BY	RE-EXAMINED BY	GRADE	DA
Radiography	III	97%	4-5-77	W. C. King			
Magnetic Particle	II	98%	8-23-76	W. C. King			
Liquid Penetrant	I	96%	10-1-75	W. C. King			
Visual Inspection	III	95%	10-21-75	W. C. King			
Welding Inspection							
Radiation Safety							
Ultrasonics	II	94%	11-25-75	W. C. King			

## EXAMINATION GRADES IN PERCENT

SUBJECT	GENERAL EXAM	SPECIAL EXAM	PRACTICAL EXAM	COMPOS
Radiography	93%	90%	93%	92%
Magnetic Particle				98%
Liquid Penetrant	100%	97%	93%	96%
Visual Inspection	100%	80%	100%	93%
Welding Inspection				
Radiation Safety				
Ultrasonics	90%	95%	100%	95%

## PERCENTILE WEIGHT

SUBJECT	GENERAL EXAM	SPECIAL EXAM	PRACTICAL EXAM	LEVEL
Radiography	.5	.3	.2	II
Magnetic Particle	.4	.3	.3	II
Liquid Penetrant	.4	.3	.3	II
Visual Inspection	.5	.4	.1	II
Welding Inspection				
Radiation Safety				
Ultrasonics			.2	II

Affidavit

My name is Charles Stokes. I am submitting this affidavit to the Nuclear Regulatory Commission (NRC) to inform them of material false statements and other evidence of activities which could compromise the quality of the Diablo Canyon nuclear power plant, if it should be turned on. The misconduct involves welding, procedure qualification tests, and plant modifications during the hot functional tests. In my professional judgment, if these issues alone are confirmed as examples of general practices, the plant could not possibly be licensed to go critical under the NRC's legal requirements in 10 C.F.R.

In fact, the practices revealed below and others I have disclosed would even flunk Bechtel's own standards. I am enclosing as Exhibit 1 portions of Bechtel's "Field Engineer Pocket Hanger Reference," Diablo Canyon Project, Bechtel Power Corporation. Bechtel's booklet is not a bad document. Although there are a few minor errors, it describes a reasonable design control and quality assurance (QA) program.

Unfortunately it was not issued on-site before I left. I obtained a copy before distribution was stopped. I can understand why Bechtel didn't want the booklet released. The plant wasn't built at all like the system described in Bechtel's own handbook. The handbook will be discussed in more detail below.

(1) In reply to PG&E's letters no. DCL-84-067 and no. DCL-84-078 concerning welding of A-325 bolts. PG&E contends that "10 supports were identified which used welded A-325 bolt design." That is highly misleading. In reality, there are many more cases where bolts have been used.



Because of inadequate documentation, welded bolts have been used and it is impossible to say whether they are A-325 or A-307 or anything else. Even QA Personnel concedes not knowing. In two specific cases, for which I can provide the support numbers, undocumented bolts were used to connect support members to structural steel.

In my opinion, PG&E's reply is so far from complete that it does not provide accurate information to the NRC concerning the use of A-325, A-307 or other bolts. The two specific supports do not even have a weld symbol describing how they were welded on the drawings. The QA inspector was not able to visually inspect the connection.

(2) A second illustration of deficient documentation for welding bolts is inadequate material traceability. Material was not stamped for traceability back to the Certificates of Compliance as required. The significance of stamping for traceability is that without this traceability there existed no methodology to ensure that the material used in many hangers, or other seismic class one structures, complied with the requirements (e.g., proper metallurgical properties).

In ANSI B31.7 chapter 10723, entitled "Materials," it is stated that "all material shall be clearly identified" by "the applicable material specification and grade, heat number, or heat code of the material, and any additional markings required to facilitate traceability of the reports of the results of all tests and examinations performed on the material." ANSI B31.7 also states that "Certificat of Compliance with the material specifications may be provided in lieu of Certified Material Test Reports unless otherwise required by the design specification." (Emphasis added)

Material traceability is only one aspect of the required traceability. In ANSI B31.7 Para. 1-727.5.3 and Para. 1-727.6, weld traceability is also required. "The welder or welding operator shall identify it as his work by applying his assigned symbol for permanent record in a manner specified by his employer. As an alternative, the employer shall keep a record of the joints and of the welders working the joints." This is also true under ASME Section IX QW-301.3, entitled "Identification of Welders and Welding Operations," which states: "Each qualified welder and welding operator shall be assigned an identifying number, letter, or symbol by the manufacturer or contractor, which shall be used to identify the work of that welder or welding operator."

In discussions with Pre-inspection Engineers, QC and QA inspectors, some of whom have worked for as long as ten years at Diablo, it is obvious that neither material nor welder traceability was maintained. All that was required was that the "Certificate of Compliance" be provided. This superficial attempt to comply with the requirements of ANSI B31.7 and ASME Section IX does not satisfy the code requirements. This is evidenced by past and present industry practice at other plants across the United States. The abuse of traceability destroys the foundation of a valid Quality Control Program -- accountability and traceability.

Since many of the pre-inspection engineers and QC, QA personnel have never before Diablo worked at a nuclear plant nor other heavy industry construction site nor read ANSI B31.1, B31.7 or ASME Section IX, they worked at Diablo under the false assumption that the work was being performed correctly, and that management was implementing all the necessary directives for them to do their work. Management did not train personnel, nor did they correct this misconception.

Having worked on other nuclear plants, I know the importance of these sections in ANSI B31.7 and ASME Section IX. At other plants almost everything in Class I systems was stamped and logged, and records were kept to insure that traceability was maintained. Per B31.7, "The marking or marking code shall be transferred to all pieces when material is cut to make more than one piece." In my experience at other plants, this was required for all Class I material except miscellaneous material, such as "gaskets, packing, seals, springs, bearings, retaining rings, washers, fluids for hangers, etc." This was not done at Diablo Canyon. The practice of using "non traceable" steel was widespread throughout the plant. At other plants shim stock was not required to be stamped, and I suppose shim stock was considered to be "etc."

B31.7 states in the case of miscellaneous items that "A list of such materials shall be furnished, and such materials do not require certified materials test reports or certificates of compliance as defined in 1-723.1.2." (Emphasis added) Management at Diablo Canyon have failed to provide the chain of documentation which is necessary under 10 C.F.R. 50 before the plant can be operational. Not only did they fail to provide an "up-to-date heat number log," but also failed to publish a list of material that did not have to meet the scrutiny of ANSI B31.7.

(3) Deficient training reinforced the problems, and perpetuated them. QA inspectors told me that their training consisted of reading ESD 223 for one week and being given a list of suggested reading. This list contained B31.7, B-31.1 and other codes. In one conversation, when I asked if the QC inspectors were required to read the suggested readings, his reply was "no, we only had to know what B31.7 was, not what it says." "I and others thought that these codes had been incorporated into ESD 223 by management." This was, and remains, a wrong assumption. The inspectors undoubtedly performed to the best of their

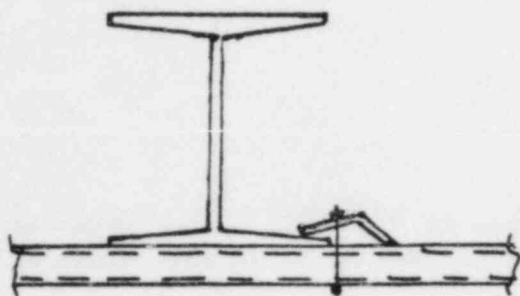
ability. However, the instruction, training, and practices necessary to adequately perform their functions were deficient. The inspectors only discovered their "wrong beliefs" through discussions with better trained, more experienced inspectors from companies other than PG&E.

(4) In letter no. DCL-84-094, PG&E states, "Pipe support number 100-111, identified for NRC review by Mr. C. Stokes, resulted in a minor modification . . . This change was made for consistency with Project Standard Practices even though analysis showed the change was not necessary to meet acceptance criteria."

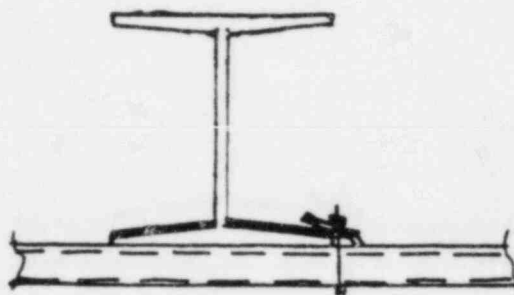
I don't know if PG&E reported other modifications performed during the hot functional testing to the NRC. I do know of at least one other support which was modified during hot functional testing. I can not give the support number here. My informant would be immediately on the "firing" line. I will supply the support number to NRC inspector Isa Yin, if the NRC supplies a list of supports to me for which they know modifications have been performed.

(5) In PG&E's answer to the intervenor's motion to reopen licensing issues on Construction Quality Assurance, "Affidavit of D.A. Rockwell, L.R. Wilson," Paragraph 3 states in part: "Since this contact is provided by the plate of the clamp to the Unistrut, the plate is not necessarily horizontal and may appear 'cockeyed.'" This statement is too incomplete to be meaningful. The use of the term "cockeyed" is not explained or supported nearly enough to support any conclusion that the clamping plates were correctly installed. If incorrectly installed, the clamp will tend to slip off the structural steel to which it is attached. See sketches below of correct installation compared to incorrect installation.





Correct. Notice Line contact at toe and heel of plate. When bolt is torqued properly, clamp should not be easily displaced.



Incorrect. Line contact but not at toe. When torqued, this tends to slip off the steel marker.

In both the examples above, the plate is "cockeyed." One is correct, and if installed correctly, should not be easily moved. On the other hand, the incorrect installation could slip easily. This fact can be checked by consulting engineering manuals from either Unistrut, Superstrut, or other brand names.

(6) In paragraph 5 as a remedy for possible slipping, PG&E states, "For support type S221, U-bolts were torqued and U-bolt nuts tack welded. For other support types, the Unistrut channel was directly welded to the beam flange." (Emphasis added) Based on my experience in the nuclear industry, the proposed fix by PG&E/Foley would do more damage than good. To my knowledge, there are no engineering documents presently available or in use that support the practice of welding Unistrut or similar material. In fact, the material type used in making "Superstrut" and similar products should not be welded. In a phone call on 3/27/84 with a Superstrut Product Engineer, I was told that Superstrut is coated with an electro-plated galvanized chromate coating (an epoxy paint) which burns when welded, giving off toxic gases. Two problems result from welding it. (1) Air quality problems for the welder and (2) the joint corrodes. The Product Engineer said he would never

advise that Superstrut be welded when used as Class I supports in a nuclear plant near the ocean. He said that the material could be destroyed in one year if exposed to adverse conditions.

(7) In reply to intervenor's Petition to reopen Construction Quality Assurance, Affidavit of H.R. Arnold, F.C. Breismesiter and R.K. Rhodes Paragraph 6. "During a planned review of existing brazing procedures for copper and stainless steel by Foley QA Personnel in September 1981, it could not be verified that stainless steel tubing PBS number MD045 had been qualified in all braze flow positions (vertical-up, vertical-down, horizontal and flat) since the procedure qualification tests performed in 1977 did not include the vertical-up flow position. This variation was properly documented on Foley Non-Conformance Report (NCR) #8802-675 in accordance with approved procedures." (Emphasis added) The statement quoted above is in direct contradiction to the first line in Para. 1 and line, Page 1. "This allegation is completely false. The procedures in question were qualified prior to their use." (Emphasis added) To correct this problem, one worker was tested. Under ANSI B31.7 and ASME Section IX, each welder must be qualified to perform the work to which he is assigned. Foley's solution does not correct the use of the procedure from 1977 to 1981 for brazing a vertical-up joint as was originally stated in the procedure. Nor does it resolve the issue as to whether the brazers before 1977 were qualified to perform work. The test of one worker does not satisfy ASME requirements that each worker be qualified unless the worker tested was the only person on-site who was assigned the brazing work. Nor do the present tests qualify old work, since past work could be considered training thus not qualifying as acceptable work. ASME Section IX requires that the welder be qualified first before work is performed. There is a reason for this,

which is to ensure that the work is performed correctly. The other point not sufficiently covered in Foley's reply is that "Neither the ASME Code nor Foley procedures require documentation of these inspections. Therefore none were documented." Nor in the statement that "ASME Section IX recognizes the function of independent mechanical test contractors such as Central Coast Lab, and does not require them to witness the actual brazing." (Paragraph 3, page 6 and 7) This is an example of Management's near-sightedness. Can they say that this documentation is not required in B31.1, B31.7, ASME Section IX, AWS D1.1-79 or 10 C.F.R? From my previous experience in the nuclear industry, it has been the practice to test and document results therefrom for welders. This would certify that the weld was made by the specific welder and that the test results were for the welds performed by that individual. These logs and records were controlled and monitored by the QA. The policies at Diablo by PG&E, Pullman, and Foley are at the opposite end of the scale from what has been typical industry practice. Where documentation was in question, other plant owners considered it good engineering practice and a good policy to go ahead and provide documentation to prevent the problem of a future question. At Diablo, just the opposite is true.

(8) In a discussion with a friend, I was shown a Discrepancy Report written against Unit #2. This document listed many anchor and smaller supports which did not have acceptable full penetration welds at the stantion to pipe and were to be reworked. The problem with this work was that there had been no process sheets issued for the removal nor had the pipe been ultrasonically tested to ensure that the minimum wall remained after grinding away the old material. The new stanchions were installed without an ultrasonic test (UT) being performed. The tests were performed seven months later. Per ASME Section IX and ANSI B31.7, the ultrasonic testing should have been conducted

at the time after removal and before new stanshions were welded in place. When ultrasonically testing this type of joint, incorrect readings are possible.

A worker who was familiar with this Discrepancy Report (DR) on Unit 2 realized the same problem might have occurred on Unit 1. I was shown a copy of a Preliminary Discrepancy Report listing about 15 supports in Unit 1 which the worker had determined had the same problem as the Unit 2 problem narrated above. I can supply the DR number on Unit 2 and the author of the Unit 1 DR. This will be supplied under similar conditions listed on a previous issue to Isa Yin.

(9) In closing and as the only exhibit to this affidavit, I have a copy of a document which was scheduled to be issued to all field engineers to aid them in their work at Diablo. It was prepared by Bechtel Power Corporation. The title of this document is Field Engineer Pocket Hanger Reference. This document was sent to the field for issuing, but was recalled under the excuse that it contained errors which needed to be corrected. I and other engineers at Diablo had copies of this document. It contains valuable information to which an engineer could refer and rely upon during his work. In truth, this document represents Bechtel policy at previous jobs. Much of it is in direct contradiction to the procedures used to build Diablo. Had it been issued many problems would have surfaced in a relatively short time. Why is this true? The document puts at finger tip location contradictory guides, providing typical industry practice in many areas, to the procedures and management directives issued at Diablo. There are minor errors in this document. However, I have reviewed it and have found it to be a valuable and handy document to have when working in the field. It should have been checked, corrected, issued and used.



Enclosed are pages 1-10 and 1-11, "Notes: Pipe Insulation Chart." In reading these two pages several points are evident which were not complied with at Diablo: (1) vapor barrier requirements; and (2) the application of a double layer of insulation on high thermal lines. In PG&E's answers to the staff concerning stress walkdown, they tried to explain away interferences by local crushing of calcium silicate. Note, this is not acceptable on Page 1-10.

Also enclosed is a copy of page 1-13, "Insulation Removal Request Flow Chart" and page 1-14, "Request for Insulation Removal." I am not aware of either of these procedures being followed at Diablo.

Also enclosed is a copy of Section 7, "Welding Instruction." On page 7-2, item 15, it is stated that there are no dihedral angle limitations for skewed T-joints. I feel this policy will cause problems by design personnel failing to consider welds shown as fillet as partial penetration groove welds unless a note specifically stated that it should be considered otherwise. I personally know many engineers will assume a fully effective throat for any weld indicated as a fillet. I suggest a test at site on this point before a decision is made on how to represent a skewed T-joint.

Also on page 7-5, see "attachment I." Either I don't understand this table or no allowance was added for the throat deduction for inadequate penetration. This last conclusion was also that of a pre-inspect engineer at Diablo Canyon.

Lastly, on pages 7-7 thru 7-10, I would like to point out the concise clarification of weld symbol terminology. Had this part of the book been in effect at Diablo, many questions would have been resolved (although many

other questionable practices would have become evident to many field personnel).

I have read the above 11-page statement and it is true and correct to the best of my knowledge and belief.

Charles C. Stokes 4/12/84  
Charles Stokes



# NOTES: PIPE INSULATION CHART

Fittings and valves are normally insulated with pre-formed sections of the same material as used on the pipe. However segmental blocks are sometimes used by wiring them on the fitting or valve and filling the gaps with asbestos cement.

Many and varied types of jackets for insulation are available. Some of these are: canvas, asphalt-saturated asbestos felt, plastic-impregnated asbestos felt, thin gage sections of galvanized steel, and integral or separate jackets of aluminum alloy or stainless steel.

The selection of the type of jacket to be used should be made with due consideration given to the following: amount of abrasion and mechanical abuse to be encountered. Weather protection for outdoor installations, fire proofing, vapor barrier requirements, cost of application, and last but not least the finished appearance.

Most insulation failures are caused by water entering through breaks in the finish, such as expansion cracks, or un-flashed openings, therefore, particular attention should be given to complete detailed specifications in regard to weatherproofing.

The usual insulating materials and jackets for heated piping and equipment allow the moisture to escape in the form of vapor. However in the medium temperature range, and where shut-downs are frequent, moisture in the insulation is not driven off and water damage is most likely to occur. For these conditions, the insulation should be thoroughly dry before applying the jacket, the surface of the pipe should be primed and painted, and corrosion-restraint wire or bands used for securing the insulation. If possible, insulation should be applied to high temperature piping while heated to insure the complete dryness of the completed installation.

The layout of insulated piping and equipment should provide adequate clearances for proper application of the insulation and also safeguard against mechanical damage during normal operation and maintenance.

Thicknesses, as shown are nominal. Actual thicknesses, which are based on "simplified thicknesses," will vary slightly from nominal in that the outer diameters of the insulation approximate the diameters of iron pipe sizes in order to obtain nesting of layers.

Weights, as shown, are for actual thicknesses and are based on densities of 11 pounds per cubic foot for 85% magnesia and calcium silicate, and 21 pounds per cubic foot for diatomaceous earth (calcined diatomaceous silica and asbestos fiber). Weights for materials with different densities may be proportioned accordingly.

At temperatures above 600°F, pipe expansion is a significant factor. For best results pipe insulation should be applied in double layers with staggered joints for all operating temperatures above 600°F. This construction prevents excessive heat losses and surface temperatures at the joints, opened by pipe expansion, thus eliminating scorched or burned jackets. This construction also eliminates the potential fire hazard of exposing jackets to higher temperatures at the joint. Double-layer, staggered-joint construction also minimizes thermal stresses in the insulation by reducing the temperature differential across each layer.

The recommended thicknesses in the above table are calculated on an economic basis for heat conservation under average operating conditions for steam generation and process piping, and assure adequate temperature control. To determine the most economical thickness for any particular installation the cost of fuel, fixed charges, need for definite temperature control, or other special conditions must be considered. If necessary, insulation thickness can be increased.

Generally, pipe insulation is furnished in 3-foot half sections for pipe sizes up to 24-inch in calcium silicate and up to 14-inch pipe size for 85 percent magnesia and diatomaceous earth. Insulation for pipe sizes larger than those mentioned is usually furnished in 3-foot long segments or curved blocks.

## LINE DESIGNATIONS (NOTES 1 THROUGH 16)

1. LINE

2. C/P - COND. FLOW RATE, IN, BY C/P & S/P  
3. S/P - SATELLITE FLOW RATE, IN, BY S/P & S/P  
4. S/P - SATELLITE FLOW RATE, IN, BY S/P & S/P

5. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY  
6. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY  
7. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

8. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

9. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

10. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

11. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

12. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

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15. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

16. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

17. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

## LINE DESIGNATIONS (ADDITIONAL NOTES)

1. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

2. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

3. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

4. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

5. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

6. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY

7. INDICATES THAT PIPE IS INSULATED OR NOT ACCORDING TO CATEGORY



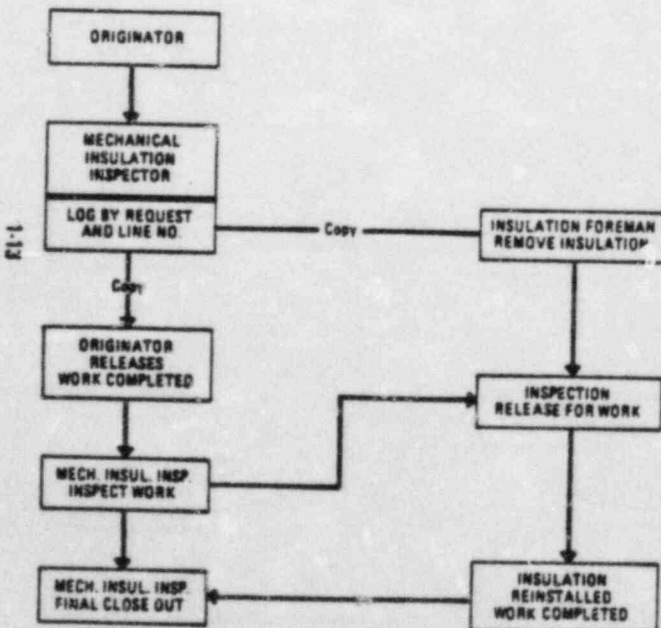
# APPENDIX E OF DCM-89

THERMAL INSULATION SPECIFICATION		THERMAL INSULATION SPECIFICATION			
CLASS	TEMP. RANGE (°F)	IN	OUT	IN	OUT
1	0-100	1"	1"	1"	1"
2	100-200	1"	1"	1"	1"
3	200-300	1"	1"	1"	1"
4	300-400	1"	1"	1"	1"
5	400-500	1"	1"	1"	1"
6	500-600	1"	1"	1"	1"
7	600-700	1"	1"	1"	1"
8	700-800	1"	1"	1"	1"
9	800-900	1"	1"	1"	1"
10	900-1000	1"	1"	1"	1"

1-12

- NOTES
- FOR CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, INSULATION SHALL BE USED FOR CLASSES 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11. INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11. INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11.
  - THE USE OF SPECIAL INSULATION IS TO BE DETERMINED BY THE PERFORMANCE OF THE INSULATION IN THE FIELD. INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11. INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11.
  - FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11, INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11. INSULATION SHALL BE USED THROUGHOUT FOR INSULATION CLASS 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, AND 11.
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## INSULATION REMOVAL REQUEST FLOW CHART



## REQUEST FOR INSULATION REMOVAL

REQUEST NUMBER \_\_\_\_\_

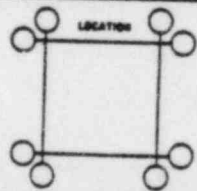
REQUEST BY \_\_\_\_\_ NAT NO. \_\_\_\_\_ EXT. \_\_\_\_\_ DEPT. \_\_\_\_\_  
 DOES LINE REQUIRE HYDRO BECAUSE OF NEW ATTACHMENT YES \_\_\_\_\_ NO \_\_\_\_\_  
 DOES LINE INVOLVE HEAT TRACING \_\_\_\_\_ CHAS. NO. \_\_\_\_\_ TRACE NO. \_\_\_\_\_  
 STAINLESS STEEL FOR CHLORIDE AND FLUORIDE TESTING REQUIRED YES \_\_\_\_\_ NO \_\_\_\_\_  
 ALL LINES REQUIRING HEAT TRACING MUST BE COORDINATED WITH ELECTRICAL DEPT.

PURPOSE \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

1-14

SHIFT NO. \_\_\_\_\_  
 AREA \_\_\_\_\_  
 BUILDING \_\_\_\_\_  
 ELEVATION \_\_\_\_\_  
 LINE NO. \_\_\_\_\_  
 HANGER NO. \_\_\_\_\_

\*ESTIMATED COMPLETION DATE \_\_\_\_\_



RELEASED FOR REMOVAL & ISSUED \_\_\_\_\_ DATE \_\_\_\_\_ EXT. \_\_\_\_\_  
 ELECTRICAL HEAT TRACE COORDINATOR \_\_\_\_\_ DATE \_\_\_\_\_ EXT. \_\_\_\_\_  
 ORIGINATOR RELEASE FOR DEINSULATION \_\_\_\_\_ DATE \_\_\_\_\_ EXT. \_\_\_\_\_  
 MECHANICAL INSULATION INSPECTOR \_\_\_\_\_ DATE \_\_\_\_\_ EXT. \_\_\_\_\_  
 INSULATION FOREMAN COMPLETED \_\_\_\_\_ DATE \_\_\_\_\_ EXT. \_\_\_\_\_

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WELDING INSTRUCTIONS  
(EXCERPTED FROM ESD-223 REV. 7)

- 1 WELDS SHALL NOT BE PEENED.
- 2 FULL PENETRATION WELDS SHALL BE FIT IN ACCORDANCE WITH THE WELD PROCEDURES DESIGNATED BY THE FIELD ENGINEER. ROOT OPENINGS USING BACKING STRAPS SHOULD BE  $1/4"$  PLUS OR MINUS  $1/16"$ . ROOT OPENING REQUIREMENTS FOR GTAW WELD PROCEDURES ARE  $1/8"$  PLUS OR MINUS  $1/32"$ .
- 3 PARTIAL PENETRATION GROOVE WELDS SHALL BE FIT IN ACCORDANCE WITH THE WELD PROCEDURE DESIGNATED BY THE FIELD ENGINEER. THE JOINT SHALL BE BROUGHT INTO AS CLOSE CONTACT AS POSSIBLE. THE GAP BETWEEN PARTS SHALL NOT EXCEED  $3/16"$ .
- 4 SQUARE GROOVE WELDS SHALL BE FIT IN ACCORDANCE WITH THE WELD PROCEDURE DESIGNATED BY THE FIELD ENGINEER. ROOT OPENING SHALL BE  $0"$  TO  $1/16"$  MAXIMUM.
- 5 FILLET WELDS SHALL BE FIT IN ACCORDANCE WITH THE WELD PROCEDURE DESIGNATED BY THE FIELD ENGINEER. PARTS TO BE JOINED SHALL BE BROUGHT INTO AS CLOSE CONTACT AS POSSIBLE. NO PACKING, SHIMMING, OR WELDING WILL BE PERMITTED TO CORRECT POOR FIT-UP OF JOINTS. IF THE SEPARATION IS  $1/16"$  OR GREATER, THE LEG OF THE FILLET SHALL BE INCREASED BY THE AMOUNT OF THE SEPARATION. THE GAP SHALL NOT EXCEED  $3/32"$ .
- 6 ALL MEMBERS SHALL BE VISUALLY PLUMB, TRUE TO LINE, AND SUBSTANTIALLY FREE FROM BENDS, TWISTS, OR EXCESSIVE GAPS.
- 7 ALL WELD PREPARATION SURFACES SHALL BE FREE FROM FOUNDRY OR MILL SCALE, OIL, RUST, SAND, SLAG, PAINT, AND ANY TYPE OF SURFACE OXIDE OR DIRT.
- 8 WELD PREPARATION SHALL BE SMOOTH AND UNIFORM. IF WELD PREPARATIONS ARE FORMED BY FLAME CUTTING, THEY SHALL BE GROUND AND DRESSED BEFORE WELDING.
- 9 UNDERCUT, EXTENDING THE LENGTH OF THE WELD, SHALL NOT EXCEED  $1/32"$  IN DEPTH. LOCAL UNDERCUT SHALL NOT EXCEED  $1/16"$  WHEN THE LENGTH OF A LOCAL UNDERCUT AREA DOES NOT EXCEED  $1/2"$  IN ANY  $6"$  LENGTH OF WELD.
- 10 FOR GROOVE AND FILLET WELDS, THE SUM OF THE DIAMETERS OF POROSITY SHALL NOT EXCEED  $3/8"$  IN ANY LINEAR INCH OF WELD AND SHALL NOT EXCEED  $3/4"$  IN ANY  $12"$  LENGTH OF WELD.
- 11 MINOR ARC STRIKES ON SUPPORT MEMBERS SHALL BE MINIMIZED. SERIOUS ARC STRIKES ON SUPPORT MEMBERS SHALL BE REMOVED AND/OR REPAIRED PRIOR TO QC ACCEPTANCE. NO ARC STRIKES ON PIPE SHALL BE PERMITTED. ARC STRIKES ON PIPE AND SEISMIC LIMITERS SHALL BE REPORTED ON A DEFICIENT CONDITION NOTICE FOR DISPOSITION PER ESD-268.
- 12 THE FILLET WELD SIZE SHALL BE AS SPECIFIED ON THE DRAWING. WHERE THE SIZE IS NOT SPECIFIED, THE FILLET SHALL BE OF THE SAME SIZE AS THE THICKNESS OF THE THINNER OF THE TWO MEMBERS BEING JOINED. AS-BUILT IS REQUIRED TO SHOW WELD SIZE.

- 13 OVERWELD: FOR EXISTING WELDS ANY AMOUNT OF OVERWELD IS ACCEPTABLE, PROVIDED DISTORTION IS NOT EXCESSIVE. FOR NEW WELDS THE MAXIMUM OVERWELD SHALL BE 50 PERCENT OR  $1/8"$ , WHICHEVER IS GREATER, PROVIDED THAT DISTORTION IS NOT EXCESSIVE. (SEE PARAGRAPH 6.8.2.4.A.) AS-BUILT IS NOT REQUIRED.
- 14 UNDERWELD: FOR EXISTING WELDS, ANY UNDERWELD IS ACCEPTABLE PROVIDED THAT THE AISC RECOMMENDED MINIMUM WELD SIZE FOR THE MATERIAL BEING WELDED IS MET. A NEW FILLET WELD IN ANY SINGULAR CONTINUOUS WELD SHALL BE PERMITTED TO UNDERRUN THE NOMINAL FILLET WELD SIZE REQUIRED BY  $1/16"$  WITHOUT CORRECTION, PROVIDED THAT THE UNDERSIZED PORTION OF THE WELD DOES NOT EXCEED 10% OF THE LENGTH OF THE WELD.

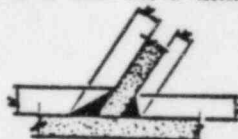
AS-BUILT IS REQUIRED TO SHOW EXISTING WELDS WHICH ARE MORE THAN 25 PERCENT UNDERSIZED AND/OR UNDERSIZE FOR MORE THAN 20 PERCENT OF THE TOTAL LENGTH OF THE WELD.

FILLET WELD SIZE TABLE

MATERIAL THICKNESS OF THICKER PART JOINED	MINIMUM SIZE OF FILLET WELD
UP TO AND INCLUDING $1/4"$	$1/8"$
OVER $1/4"$ THROUGH $1/2"$	$3/16"$
OVER $1/2"$ THROUGH $3/4"$	$1/4"$
OVER $3/4"$ THROUGH $1-1/2"$	$5/16"$
OVER $1-1/2"$ THROUGH $2-1/4"$	$3/8"$
OVER $2-1/4"$ THROUGH $6"$	$1/2"$
OVER $6"$	$5/8"$

15 FILLET WELDS ON SKEWED T-JOINTS

- 1 THERE ARE NO DIEDRAL ANGLE LIMITATIONS
- 2 THE SIZE OF FILLET WELDS ON SKEWED T-JOINTS SHALL BE DETERMINED BY MEASURING THE NOMINAL LEG, AS FOLLOWS:

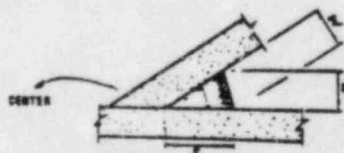


W = NOMINAL WELD LEG

NOTE: WELD SURFACES MUST BE FLAT ON CONVEX.

16 PARTIAL PENETRATION WELDS

- 1 WHEN VERIFYING THE SIZE OF PARTIAL PENETRATION BEVEL WELDS, THE FACE MUST BE AT LEAST FLUSH WITH THE FACE OF THE BEVELED PIECE.
- 2 QC FIT-UP INSPECTION IS REQUIRED ON PARTIAL PENETRATION GROOVE WELDS DESIGNED AT SKEWED T-JOINTS. THE SIZE OF PARTIAL PENETRATION GROOVE WELDS, WHICH HAVE BEEN DESIGNED TO BE INSTALLED AT SKEWED T-JOINTS, SHALL BE DETERMINED BY MEASURING THE FACE OF THE WELD, AS FOLLOWS:



W = NOMINAL WELD LEG

S = SPECIFIED SIZE

USE W MINIMUM IN DETERMINING WELD SIZE.

THE ACCEPTABILITY OF THE WELD SIZE SHALL BE DETERMINED BY USING THE CHART IN ATTACHMENT I.

17 FLARE-BEVEL WELDS WHICH ARE FORMED BY AT LEAST ONE PIECE OF TUBE STEEL SHALL HAVE THE SIZE DETERMINED AS FOLLOWS:

- 1 WHEN NO SIZE IS SPECIFIED ON THE DRAWING, THE FACE OF THE WELD SHALL BE AT LEAST FLUSH WITH THE FACE OF THE TUBE STEEL BEING WELDED
- 2 WHEN THE SIZE OF THE WELD IS SPECIFIED ON THE DRAWING, THE SIZE OF THE WELD SHALL BE DETERMINED BY MEASURING THE FACE OF THE WELD AS FOLLOWS:



W = NOMINAL WELD FACE

S = SPECIFIED SIZE

THE ACCEPTABILITY OF THE WELD SIZE SHALL BE DETERMINED BY USING THE CHART IN ATTACHMENT J.

[illegible]

FLANGE JOINT WELDING SYMBOL	STANDARD INTERMITTENT FILLET WELDING SYMBOL	SINGLE V GROOVE WELDING SYMBOL INDICATING ROOT PENETRATION
<b>FLANGE JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>STANDARD INTERMITTENT FILLET WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>SINGLE V GROOVE WELDING SYMBOL INDICATING ROOT PENETRATION</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>SPOT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>SEAM WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>WELDING SYMBOLS FOR COMBINED WELDS</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>BASIC JOINTS - IDENTIFICATION OF ARROW SIDE AND OTHER SIDE OF JOINT</b>	<b>SEAM JOINT</b>	<b>DOUBLE BUTT JOINT</b>

7-10

FLANGE JOINT WELDING SYMBOL	STANDARD INTERMITTENT FILLET WELDING SYMBOL	SINGLE V GROOVE WELDING SYMBOL INDICATING ROOT PENETRATION
<b>FLANGE JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>STANDARD INTERMITTENT FILLET WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>SINGLE V GROOVE WELDING SYMBOL INDICATING ROOT PENETRATION</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>SPOT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>SEAM WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>WELDING SYMBOLS FOR COMBINED WELDS</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.	<b>DOUBLE BUTT JOINT WELDING SYMBOL</b> The arrow side of the weld is indicated by the arrow pointing to the weld line. The length of the weld is indicated by the dimension line.
<b>BASIC JOINTS - IDENTIFICATION OF ARROW SIDE AND OTHER SIDE OF JOINT</b>	<b>SEAM JOINT</b>	<b>DOUBLE BUTT JOINT</b>

7-8

LOCATION SIGNIFICANCE	FILLET	PLUS OR MINUS	NO PROJECTION	SEAM	NO BACKING	SURFACING	SEAM (NO BEADED JOINT)	FLANGE JOINT
<b>ARROW SIDE</b>								
<b>OTHER SIDE</b>								
<b>NO ARROW SIDE (NO OTHER SIDE) SIGNIFICANCE</b>								
<b>SUPPLEMENTARY SYMBOLS USED WITH WELDING SYMBOLS</b>								
<b>CORNER CORNER SYMBOL</b>			<b>WELL-ALL AROUND SYMBOL</b>			<b>MELT THERM SYMBOL</b>		
<b>JOINT WITH BACKING</b>			<b>JOINT WITH SPACER</b>			<b>JOINT WITH SPACER</b>		

7-9



GOVERNMENT ACCOUNTABILITY PROJECT

Institute for Policy Studies  
1901 Que Street, N.W., Washington, D.C. 20009

ATTACHMENT  
3

(202) 234-9382

March 2, 1984

Mr. Thomas Bishop  
Division Director  
U.S. N.R.C.  
Region Five  
1450 Maria Lane, Ste 210  
Walnut Creek California 94596

Re: PACIFIC GAS AND ELECTRIC (Diablo Canyon Nuclear Power  
Plant, Unit 1), Dkt. No. 50-275

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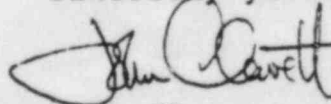
Dear Mr. Bishop:

Enclosed with this letter is a copy of a petition filed with the Commission pursuant to 10 C.F.R. 2.206 on March 1, 1984, together with Attachments 1 through 17 thereto.

Also enclosed for your use, bound separately in this package, are three documents that were not included in the petition filed on February 2, 1984: Exhibit 4 to Attachment 2, and two Discrepancy Reports inadvertently omitted from Attachment 7.

Best wishes in your continuing investigation.

Sincerely yours,



John Clewett

Enclosures: A/S