TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
2. MAIN STEAM LINE ISOLATION (Continued)		
g. Main Steam Line Tunnel Δ Temp. - High	≤ 101°F**	≤ 104°F**
h. Manual Initiation	NA	NA
3. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level - Low Low, Level 2	≥ -41.6 inches*	≥ -43.8 inches
b. Drywell Pressure - High	≤ 1.73 psig	≤ 1.93 psig
c. Fuel Handling Area Ventilation Exhaust Radition - High High	≤ 2.0 mR/hr**	≤ 4.0 mR/hr**
d. Fuel Handling Area Pool Sweep Exhaust Radiation - High High	≤ 18 mR/hr**	≤ 35 mR/hr**
e. Manual Initiation	NA	NA
4. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 79 gpm	≤ 89** gpm
b. Δ Flow Timer	≤ 45 seconds	≤ 57 seconds
c. Equipment Area Temperature - High	120	126
1. RWCU Hx Room	≤ ~~124~~°F	≤ ~~130~~°F
2. RWCU Pump Rooms	≤ ~~174~~°F 170 °F	≤ ~~180~~°F 176 °F
3. RWCU Valve Nest Room	≤ ~~139~~°F 135 °F	≤ ~~145~~°F 141 °F
4. RWCU Demin. Rooms	≤ 139°F	≤ 145°F
5. RWCU Rec. Tank Room	≤ 139°F	≤ 145°F
6. RWCU Demin. Valve Room	≤ 135°F	≤ 141°F
d. Equipment Area Δ Temp. - High		
1. RWCU Hx Room	≤ 65°F	≤ 66°F
2. RWCU Pump Rooms	≤ 115°F	≤ 118°F
3. RWCU Valve Nest Room	≤ 70°F	≤ 73°F
4. RWCU Demin Rooms	≤ 70°F	≤ 73°F
5. RWCU Rec. Tank Room	≤ 70°F	≤ 73°F
6. RWCU Demin. Valve Room	≤ 71°F	≤ 74°F

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TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
4. REACTOR WATER CLEANUP SYSTEM ISOLATION (Continued)		
e. Reactor Vessel Water Level - Low Low, Level 2	≥ -41.6 inches*	≥ -43.8 inches
f. Main Steam Line Tunnel Ambient Temperature - High	≤ 185°F**	≤ 191°F**
g. Main Steam Line Tunnel Δ Temp. - High	≤ 101°F**	≤ 104°F**
h. SLCS Initiation	NA	NA
i. Manual Initiation	NA	NA
5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION		
a. RCIC Steam Line Flow - High	≤ 363" H_2O	≤ 371" H_2O
b. RCIC Steam Supply Pressure - Low	≥ 60 psig	≥ 53 psig
c. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
d. RCIC Equipment Room Ambient Temperature - High	≤ 185 ~~189~~°F**	≤ 191 ~~195~~°F**
e. RCIC Equipment Room Δ Temp. - High	≤ 125°F**	≤ 128°F**
f. Main Steam Line Tunnel Ambient Temperature - High	≤ 185°F**	≤ 191°F**
g. Main Steam Line Tunnel Δ Temp. - High	≤ 101°F**	≤ 104°F**
h. Main Steam Line Tunnel Temperature Timer	≤ 30 minutes	≤ 30 minutes
i. RHR Equipment Room Ambient Temperature - High	≤ 165 ~~169~~°F**	≤ 171 ~~175~~°F**
j. RHR Equipment Room Δ Temperature - High	≤ 99 ~~105~~°F**	≤ 102 ~~108~~°F**
k. RHR/RCIC Steam Line Flow - High	≤ 145" H_2O	≤ 160" H_2O

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Continued)		
1. Manual Initiation	NA	NA
m. Drywell Pressure-High (ECCS Division 1 and Division 2)	≤ 1.89 psig	≤ 1.94 psig
6. RHR SYSTEM ISOLATION		
a. RHR Equipment Room Ambient Temperature - High	≤ ~~169~~ 165°F**	≤ ~~175~~ 171°F**
b. RHR Equipment Room Δ Temperature - High	≤ ~~105~~ 99°F**	≤ ~~108~~ 102°F**
c. Reactor Vessel Water Level - Low, Level 3	≥ 11.4 inches*	≥ 10.8 inches
d. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	≤ 135 psig	≤ 150 psig
e. Drywell Pressure - High	≤ 1.73 psig	≤ 1.93 psig
f. Manual Initiation	NA	NA

* See Bases Figure B 3/4 3-1.

** Initial setpoint. Final setpoint to be determined during startup test program. Any required change to this setpoint shall be submitted to the Commission within 90 days of test completion.

2. (Partial Resolution of MP&L P/L Item No. 306)

SUBJECT: Containment and Drywell Isolation Valves Technical Specification Table 3.6.4-1.

DESCRIPTION OF CHANGE: Revisions to Technical Specification Table 3.6.4-1 are proposed to add four check valves and one locked-closed drain valve to the Table. The valves to be added are E61-F002A, E61-F002B, E61-F004A, E61-F004B, and G41-F265. (Page 3/4 6-41).

JUSTIFICATION: This change proposes the addition of four check valves of the Combustible Gas Control System to Table 3.6.4.1. Two of these check valves, E61-F002A and B, are located on the drywell purge compressor lines (one per line). The remaining check valves, E61-F004A and B, are located on the post-LOCA drywell vacuum breaker line. In light of the fact that there are no inboard isolation valves provided for these lines, these check valves perform isolation functions as backups to the outboard isolation valves presently existing in those lines. Inclusion of these check valves in Table 3.6.4-1 because of their backup isolation functions is therefore, considered appropriate.

It is also proposed to add G41-F265, a normally locked closed drain valve, to the table. This valve is an upper containment pool drain system valve that is only opened during a refueling outage. Because this valve is on a line that penetrates the drywell, inclusion of this valve in the table is considered appropriate.

SIGNIFICANT HAZARDS CONSIDERATION:

The proposed changes to the Technical Specification have been evaluated and determined to involve no significant hazards, as defined in 10 CFR 50.92, in that the proposed changes to include five additional valves represent additional controls not presently included in the Technical Specifications.

It is concluded that the proposed changes do not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated; or

2. Create the possibility of a new or different kind of accident from any accident previously evaluated; or

3. Involve a significant reduction in a margin of safety.

Therefore, the proposed changes do not involve a significant hazards consideration.

TABLE 3.6.4-1 (Continued)

CONTAINMENT AND DRYWELL ISOLATION VALVES

SYSTEM AND VALVE NUMBER		PENETRATION NUMBER
b. Drywell		
LPCI "A"	E12-F041A	313(I)
LPCI "B"	E12-F041B	314(I)
LPCI "B"	E12-F236	314(O)
CRD to Recirc. Pump A Seals	B33-F013A	326(I)
CRD to Recirc. Pump A Seals	B33-F017A	326(O)
Instrument Air	P53-F008	~~327~~ 335 (I)
Standby Liquid Control	C41-F007	328(I)
Standby Liquid Control	C41-F006	328(O)
Cont. Cooling Water Supply	P42-F115	329(I)
Plant Service Water Supply	P44-F075	332(I)
Condensate Flush Conn.	B33-F204	333(I)
Condensate Flush Conn.	B33-F205	333(O)
CRD to Recirc. Pump B Seals	B33-F013B	346(I)
CRD to Recirc. Pump B Seals	B33-F017B	346(O)
Service Air	P52-F196	363(I)
Cont. Leak Rate Test Inst.	M61-F021	438A(I)
Cont. Leak Rate Sys.	M61-F020	438A(O)
BLIND FLANGES		
Cont. Leak Rate Sys.	NA	40(I)(O)
Cont. Leak Rate Sys.	NA	82(I)(O)
Containment Leak Rate System	NA	343(I)(O)
COMBUSTIBLE GAS CONTROL	E61-F002A	339 (O)
COMBUSTIBLE GAS CONTROL	E61-F002B	338 (O)
COMBUSTIBLE GAS CONTROL	E61-F004A	340 (O)
COMBUSTIBLE GAS CONTROL	E61-F004B	340 (O)
UPPER CONTAINMENT POOL DRAIN	G41-F265	342 (O)

3. (MP&L P/L Item No. 329)

SUBJECT: Accident Monitoring Instrumentation Applicable Operational Conditions, Technical Specification 3.3.7.5 and Table 3.3.7.5-1.

DESCRIPTION OF CHANGE: Revisions to the subject Technical Specification and Table are proposed to reflect required operational conditions for the accident monitoring instrumentation.

1. Technical Specification 3.3.7.5 should be revised to require reference to Table 3.3.7.5-1 for determining the operational condition applicable to each accident monitoring instrument. (Page 3/4 3-69).

2. Table 3.3.7.5-1 should be revised to incorporate an "Applicable Operational Conditions" heading and the operational conditions for each instrument. Also the instrument titles for Items 13 through 18 of Table 3.3.7.5-1 should be changed to reflect the specific type of monitor. (Page 3/4 3-70).

3. The ACTION statement number for item number 2 on Table 3.3.7.5-1 should be changed from Action Statement 80 to 82. (Page 3/4 3-70).

4. Action 80, parts a and b, of Table 3.3.7.5-1 should be revised to include a requirement to be in cold shutdown within 24 hours of achieving hot shutdown. (Page 3/4 3-71).

5. A new two part action statement, ACTION 82, should be incorporated that reflects the requirements of ACTION 80 a and b for Operational Conditions 1, 2, & 3 and meets the intent of ACTION 81 (except for the reporting requirement) for Operational Conditions 4 & 5. (Page 3/4 3-71).

6. Table 4.3.7.5-1 should be revised to reflect the instrument title changes in Item 13 through 18 to indicate the specific type of monitor (e.g., exhaust radiation). (Page 3/4 3-72).

JUSTIFICATION: Technical Specification 3.3.7.5 presently requires accident monitoring instrumentation to be operable in Conditions 1 and 2. However, a review of Chapter 15 of the Grand Gulf FSAR indicates that the accident monitoring instruments should be required to be operable in other operational conditions as well. In order to determine the appropriate operational condition for each of the 18 accident monitors, an evaluation was performed using Appendix 15A of the FSAR as a basis.

Appendix 15A entitled, "Plant Nuclear Safety Operational Analysis (NSOA)," is a comprehensive, total plant, system-level, qualitative Failure Modes and Effects Analysis (FMEA), relative to all Chapter 15 events and includes the protective sequences used to accommodate these events and their effects as well as the systems involved in the protective actions. The information

contained in Appendix 15A was used to aid in determining operational conditions for the accident monitoring instruments that would be consistent with the plant safety analyses. For those accident monitors which could not be directly correlated to events in Appendix 15A, the applicable operational conditions were obtained by comparison to similar instruments and good engineering judgement.

The reactor vessel water level accident monitor is unique in comparison to the other non-radiation monitors in that it is required to be operable in all operating conditions rather than only in conditions 1, 2 and 3. The present action statement no longer adequately addresses an inoperable reactor water level monitor. Action 82 is proposed specifically to address the reactor water level monitor. It is written in two parts; one which requires shutdown and cooldown from conditions 1, 2 or 3 and a second which requires either restoring the instrument to an operable status or initiating an alternate method of monitoring the level, when in conditions 4 or 5. The new action statement is consistent with both of the present action statements in Table 3.3.7.5-1.

SIGNIFICANT HAZARDS CONSIDERATION:

The incorporation of the proposed changes into the Technical Specifications will ensure that the accident monitoring instrumentation will be required to be operable during conditions that are consistent with the events described in Chapter 15 of the Grand Gulf FSAR.

MP&L considers the proposed changes conservative in terms of safety, in that they constitute an additional limitation, not presently included in the Technical Specifications. The proposed changes are not considered to:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated; or

2. Create the possibility of a new or different kind of accident from any accident previously evaluated; or

3. Involve a significant reduction in a margin of safety.

Thus, the proposed change has been evaluated and determined to involve no significant hazards consideration, as defined in 10 CFR 50.92.

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.7.5 The accident monitoring instrumentation channels shown in Table 3.3.7.5-1 shall be OPERABLE.

APPLICABILITY: ~~OPERATIONAL CONDITIONS 1 and 2.~~ AS SHOWN IN TABLE 3.3.7.5-1

ACTION:

With one or more accident monitoring instrumentation channels inoperable, take the ACTION required by Table 3.3.7.5-1.

SURVEILLANCE REQUIREMENTS

4.3.7.5 Each of the above required accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.7.5-1.

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	APPLICABLE OPERATIONAL CONDITIONS	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
1. Reactor Vessel Pressure	1,2,3	2	1	80
2. Reactor Vessel Water Level	1,2,3,4,5	2	1	82 ~~80~~
3. Suppression Pool Water Level	1,2,3	2	1	80
4. Suppression Pool Water Temperature	1,2,3	6, 1/sector	6, 1/sector	80
5. Drywell/Containment Differential Pressure	1,2,3	2	1	80
6. Drywell Pressure	1,2,3	2	1	80
7. Drywell and Control Rod Drive Cavity Temperature	1,2,3	2 (each)	1 (each)	80
8. Containment Hydrogen Concentration Analyzer and Monitor	1,2,3	2	1	80
9. Drywell Hydrogen Concentration Analyzer and Monitor	1,2,3	2	1	80
10. Containment Pressure (wide and narrow range)	1,2,3	2 (each)	1 (each)	80
11. Containment Air Temperature	1,2,3	2	1	80
12. Safety/Relief Valve tail Pipe Pressure Switch Position Indicators	1,2,3	1/valve	1/valve	80
13. Containment/Drywell Area RADIATION Monitors	1,2,3,4,5	2#	1#	81
14. Containment Ventilation EXHAUST RADIATION Monitor	1,2,3,4,5	1	1	81
15. Off-gas and Radwaste Bldg. Ventilation EXHAUST RADIATION Monitor	1,2,3,4,5	1	1	81
16. Fuel Handling Area Ventilation EXHAUST RADIATION Monitor	1,2,3,4,5	1	1	81
17. Turbine Bldg. Ventilation EXHAUST RADIATION Monitor	1,2,3	1	1	81
18. Standby Gas Treatment System A & B Exhaust RADIATION Monitors	*	1/each	1/each	81

#Each for containment and drywell.

* WHEN ITS ASSOCIATED TRAIN OF THE STANDBY GAS TREATMENT SYSTEM IS REQUIRED OPERABLE (REF 3.6.6.3)

TABLE 3.3.7.5-1 (Continued)
ACCIDENT MONITORING INSTRUMENTATION

ACTION STATEMENTS

ACTION 80 -

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours. AND BE IN COLD SHUTDOWN WITHIN THE NEXT 24 HOURS.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours. AND BE IN COLD SHUTDOWN WITHIN THE NEXT 24 HOURS

ACTION 81 -

With the number of OPERABLE accident monitoring instrumentation channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or:

a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and

b. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

ACTION 82 FOR OPERATIONAL CONDITIONS 1, 2, 3

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours. AND BE IN COLD SHUTDOWN WITHIN THE NEXT 24 HOURS.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours. AND BE IN COLD SHUTDOWN WITHIN THE NEXT 24 HOURS

FOR OPERATIONAL CONDITIONS 4, 5

With the number of OPERABLE accident monitoring instrumentation channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or Initiate the preplanned alternate method of monitoring the appropriate parameter(s).

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Reactor Vessel Pressure	M	R
2.	Reactor Vessel Water Level	M	R
3.	Suppression Pool Water Level	M	R
4.	Suppression Pool Water Temperature	M	R
5.	Drywell/Containment Differential Pressure	M	R
6.	Drywell Pressure	M	R
7.	Drywell and Control Rod Cavity Temperature	M	R
8.	Containment Hydrogen Concentration Analyzer and Monitor	NA	M*
9.	Drywell Hydrogen Concentration Analyzer and Monitor	NA	M*
10.	Containment Pressure	M	R
11.	Containment Air Temperature	M	R
12.	Safety/Relief Valve Tail Pipe Pressure Switch Position Indicators	M	R
13.	Containment/Drywell Area RADIATION Monitors	M	R**
14.	Containment Ventilation EXHAUST RADIATION Monitor	M	R
15.	Off-gas and Radwaste Bldg. Ventilation EXHAUST RADIATION Monitor	M	R
16.	Fuel Handling Area Ventilation EXHAUST RADIATION Monitor	M	R
17.	Turbine Bldg. Ventilation EXHAUST RADIATION Monitor	M	R
18.	Standby Gas Treatment System A & B Exhaust RADIATION Monitors	M	R

*Using sample gas containing:

a. One volume percent hydrogen, remainder nitrogen.

b. Four volume percent hydrogen, remainder nitrogen.

**The CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10R/hr and a one point calibration check of the detector below 10R/hr with an installed or portable gamma source.