

DUKE POWER COMPANY

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HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

TELEPHONE
(704) 373-4531

April 9, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: Catawba Nuclear Station
Docket Nos. 50-413 and 50-414

Dear Mr. Denton:

Section 8.4.4 of the Catawba Safety Evaluation Report discusses Open Item 13, Power Lockout to Motor-Operated Valves. As discussed in this section, the Staff requested that Duke Power identify the means used to lock out power to those valves for which power is removed outside the control room.

The following valves have been identified as requiring power lockout (from outside the control room) in order to meet BTP ICSB 18 (PSB):

NI 54A	CA 2
NI 65B	CA 7A
NI 76A	CA 9B
NI 88B	CA 11A

Each of these electrically operated valves will have their circuit breakers padlocked to assure that they remain in the desired position. Proposed changes to the Catawba Proof and Review Technical Specifications are attached.

Very truly yours,

H.B. Tucker

Hal B. Tucker

ROS/php

Attachment

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

NRC Resident Inspector
Catawba Nuclear Station

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Mr. Harold R. Denton, Director
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Page 2

cc: Mr. Robert Guild, Esq.
Attorney-at-Law
P. O. Box 12097
Charleston, South Carolina 29412

Palmetto Alliance
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Columbia, South Carolina 29205

Mr. Jesse L. Riley
Carolina Environmental Study Group
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EMERGENCY CORE COOLING SYSTEMS

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SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 31 days and within 6 hours after each solution volume increase of greater than or equal to (1% of tank volume) by verifying the boron concentration of the accumulator solution;
- c. At least once per 31 days when the Reactor Coolant System pressure is above 2000 psig by verifying that power ~~to the isolation valve operator is disconnected by removal of the breaker from the circuit;~~ and
- d. At least once per 18 months by verifying that each cold leg injection accumulator isolation valve opens automatically under each of the following conditions:
 - 1) When an actual or a simulated Reactor Coolant System pressure signal exceeds the P-11 (Pressurizer Pressure Block of Safety Injection) Setpoint, and
 - 2) Upon receipt of a Safety Injection test signal.

4.5.1.1.2 Each Cold Leg Injection Accumulator System water level and pressure channel shall be Demonstrated OPERABLE:

- a. At least once per 31 days by the performance of an ANALOG CHANNEL OPERATIONAL TEST, and
- b. At least once per 18 months by the performance of a CHANNEL CALIBRATION.

is removed from the valve operators on valves NI 54A, NI 65B, NI 76A and NI 88B and that the respective circuit breakers are padlocked.

SURVEILLANCE REQUIREMENTS (Continued)

- 3) Verifying that each non-automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in its correct position; and
 - 4) Verifying that each automatic valve in the flow path is in the fully open position whenever the Auxiliary Feedwater System is placed in automatic control or when above 10% RATED THERMAL POWER.
 - 5) Verifying that the isolation valves in the auxiliary feedwater pump suction lines are open, ~~and~~ that power is removed from the valve operators on valves CA-2, CA-7A, CA-9B, and CA-11A *and that the respective circuit breakers are padlocked.*
- b. At least once per 18 months during shutdown by:
- 1) Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of an Auxiliary Feedwater Actuation test signal, and
 - 2) Verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of an Auxiliary Feedwater Actuation test signal.
 - 3) Verifying that the valve in the suction line of each auxiliary feedwater pump from the Nuclear Service Water System automatically actuates to its full open position within less than or equal to 10 seconds on a Loss-of-Suction test signal.

4.7.1.2.2 An auxiliary feedwater flow path to each steam generator shall be demonstrated OPERABLE following each COLD SHUTDOWN of greater than 30 days prior to entering MODE 2 by verifying normal flow to each steam generator.