

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)					PAGE (3)	
JAMES A. FITZPATRICK NUCLEAR POWER PLANT										0 5 0 0 0 3 3 3					1 OF 2	
TITLE (4)																

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)				
03	04	84	84	007	00	04	04	84						05000				
														05000				

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 0 0	20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.406(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)		
NAME	TELEPHONE NUMBER	
	AREA CODE	
ROBERT T. LIENO, MAINTENANCE SUPERINTENDENT	3 1 5	3 4 2 - 3 8 4 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S B	I S V	R 3 4 4	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Prior to a scheduled modification of Main Steam Isolation Valves (MSIV) 29AOV-80D and 29AOV-86D, a Local Leak Rate Test (LLRT) which checked the leak rate of both valves simultaneously, was performed. The LLRT result exceeded the Limits of Technical Specification Table 3.7-1. Upon inspection of 29AOV-80D it was noticed that the seat had developed a small cut. The valve seat was machined and the valves were reassembled. A LLRT was performed upon completion of valve maintenance, with test results within Technical Specification limits. A significant hazard to the health and safety of the public did not exist because the out-board MSIV in that line was in good condition.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3			LER NUMBER (6)			PAGE (3)		
				YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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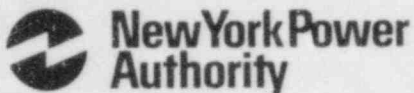
TEXT (If more space is required, use additional NRC Form 366A's) (17)

On March 8, 1984 a combined Local Leak Rate Test (LLRT) was performed on Main Steam Isolation Valves, (MSIV) 29A0V-80D and 29A0V-86D, prior to a scheduled modification. The result of this LLRT was a combined leak rate of 1018 SCFD for both MSIV's. This is in excess of the Technical Specification, Table 3.7-1 value of 276 SCFD.

The valves were disassembled and a visual examination of all valve components was performed. The only noticeable defect was a small cut in the valve seat of 29A0V-80D. This was removed by machining the seat. Both valves were also modified and re-assembled. A LLRT was performed upon completion of valve maintenance and leak rate test results were within the allowable Technical Specification limits (result: 143.5 SCFD).

The cause for this leak test failure appears to be minor seating surface degradation and is viewed as an isolated occurrence. The A main steam line valves were tested during this outage to establish that the leakage problem was not common to the other MSIV's. These valves passed the LLRT at 10.07 SCFD. Combined leakage for these valves during the 1983 refueling outage was 14.76 SCFD indicating no degradation had occurred. Since one valve in the D line showed little sign of deterioration and a common degradation was ruled out, a significant hazard to the public health and safety did not exist.

James A. FitzPatrick
Nuclear Power Plant
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Lycoming, New York 13093
315 342.3840



Corbin A. McNeill, Jr.
Resident Manager

April 4, 1984
JAFP 84-0366

Document Control Desk
United States Nuclear Regulatory Commission
Washington, DC 20555

REFERENCE: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 84-007-00

Dear Sir:

We have enclosed the referenced Licensee Event Report in accordance with 10CFR50.73.

If there are any questions concerning this report, please contact Mr. Robert Liseno at 315-342-3840, extension 220.

Very truly yours,

by dr. R. McNeill
CORBIN A. MCNEILL, JR.
RESIDENT MANAGER

CAM:RTL:nan
Enclosure

CC: Regional Administrator (1)
INPO Records Center, Atlanta, Ga. (1)
Internal Power Authority Distribution
NRC Resident Inspector
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LER/OR File

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