



**Wisconsin Electric** POWER COMPANY  
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

April 5, 1984

Mr. J. G. Keppler, Regional Administrator  
Office of Inspection and Enforcement,  
Region III  
U. S. NUCLEAR REGULATORY COMMISSION  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NOS. 50-266 AND 50-301  
REPORT ON IE BULLETIN NO. 82-02  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In IE Bulletin No. 82-02 dated June 2, 1982, the NRC notified PWR licensees of incidents of severe degradation of threaded fasteners in closures of the reactor coolant pressure boundary of various PWR nuclear plants. The bulletin instructed licensees to perform specific actions concerning threaded fasteners including an augmented inservice inspection of these threaded fasteners, specifically, bolting less than or equal to two inches in diameter. As required by Action Item 4 of IE Bulletin No. 82-02, this report on Action Items 1 and 2 is submitted.

As committed to in our response dated August 11, 1982, Action Item 1 has been completed. Point Beach Maintenance mechanics training is governed by Maintenance Training Qualification Guides which were upgraded in November 1982 in the area of Repairer I (TQ9.1) training. The additional practical factors and knowledge factors required by this qualification procedure address those items discussed under Action Item 1. The quality assurance measures for fastener lubricants and injection sealant compounds were previously discussed in our August 11 response.

Table 1 is a summary of bolting examination information gathered during the refueling outages of Point Beach Units 1 and 2 subsequent to issuance of IE Bulletin No. 82-02. The bolting examinations that were conducted were in accordance with the commitments outlined in our August 11 response.

Mr. J. G. Keppler

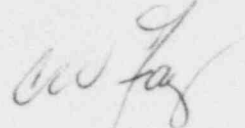
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The results of the remaining bolting examinations be conducted will be on file with the permanent inservice inspection records at Wisconsin Electric. These results will be available for review if requested.

This fulfills the required written response to IE Bulletin No. 82-02.

Very truly yours,



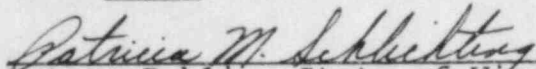
Vice President-Nuclear Power

C. W. Fay

Attachment

Copies to NRC Resident Inspector  
NRC Document Control, Washington, D. C.  
(w/original)

Subscribed and sworn to before me  
this 5th day of April 1984.

  
Notary Public, State of Wisconsin

My Commission expires May 4, 1986.

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[illegible]

## BOLTING INFORMATION

CLOSURE	NO BOLT	STUD OR BOLT MATL	NUT MATL	BOLTING SIZE	EXAMINED INPLACE/REMOVED	PAST LEAKAGE (YEAR)	EXAMINATION		EVIDENCE OF DEGRADATION
							METHOD	DATE	
Check Valve SI-842B Body-to-Cap	16	SA-193 GR-B7	SA-194 GR-2H	1-5/8"					
Check Valve SI-867A Body-to-Cap	16	SA-193 GR-B7	SA-194 GR-2H	1-5/8"	Inplace (U1)		VT-1	11/82 (U1)	None
Check Valve SI-867B Body-to-Cap	16	SA-193 GR-B7	SA-194 GR-2H	1-5/8"	Inplace (U1)		VT-1	11/82 (U1)	None
Check Valve SI-853A Body-to-Cap	12	SA-193 GR-B7	SA-194 GR-2H	1-5/8"					In 1981, one stud shows evidence of localized corrosion wastage.
Check Valve SI-853B Body-to-Cap	12	SA-193 GR-B7	SA-194 GR-2H	1-5/8"	Inplace (U2)		VT-1	4/83 (U2)	None
Check Valve SI-853C Body-to-Cap	12	SA-193 GR-B7	SA-194 GR-2H	1-5/8"					
Check Valve SI-853D Body-to-Cap	12	SA-193 GR-B7	SA-194 GR-2H	1-5/8"					

- NOTES: (1) Westinghouse specification No. BDS70041AL  
 (2) Replaced gasket  
 (3) Seal welded body-to-bonnet connection and lubricated and retightened the bolting.  
 (4) For each case of leakage, a visual exam was performed of connection after repairs to verify leaktight integrity at normal operating pressure.